

A China's Multi-purpose SMR-ACP100 Design and Project Progress

Dr. Danrong Song
Nuclear Power Institute of China
China

31 August 2022



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Meet the Presenter

Dr. Danrong SONG is the chief designer at the Nuclear Power Institute of China. Dr. Song earned his Bachelor of Science Degree in Mechanical Engineering in 1991, his Master's Degree of Business Administration in 2003, and his Doctoral Degree in Nuclear Science and Engineering from the Nuclear Power Institute of China in 2009. Currently, he works as the Chief Designer of the SMR demonstration project ACP100 in China. Dr. Song specializes in small and medium reactor overall design, isotopic production reactor overall design, seawater desalination, and low temperature nuclear heating plant feasibility studies.



Contents

- Background*
- Introduction of ACP100*
- Design Parameters of ACP100*
- Technical Aspects*
- Safety and Licensing Strategy*
- Testing & Verification*
- Demonstration Project of ACP100*

Background

- (1-1) What is a Small Modular Reactor (SMR) ?
 - SMR is a one kind of newer generation reactor designed to generate electric power up to 300 MW, whose components and systems can be shop fabricated and then transported as modules to the sites for installation.

Background

- (1-2) What is a Small Modular Reactor (SMR) ?
 - By adopting modular designs and construction concepts, the SMR is a passive safety technology.
 - Can reach high power by several modular combination.
 - Can be used in different places and in different conditions.

Background

- (2) Challenges in Increasing Large Nuclear Power Plant Power
 - Industry capacity and transportation
 - Marginal effect on the economy by increasing power
 - Difficulty with application of passive technology
 - Huge overnight investment
 - Not flexible for alternative use

Background

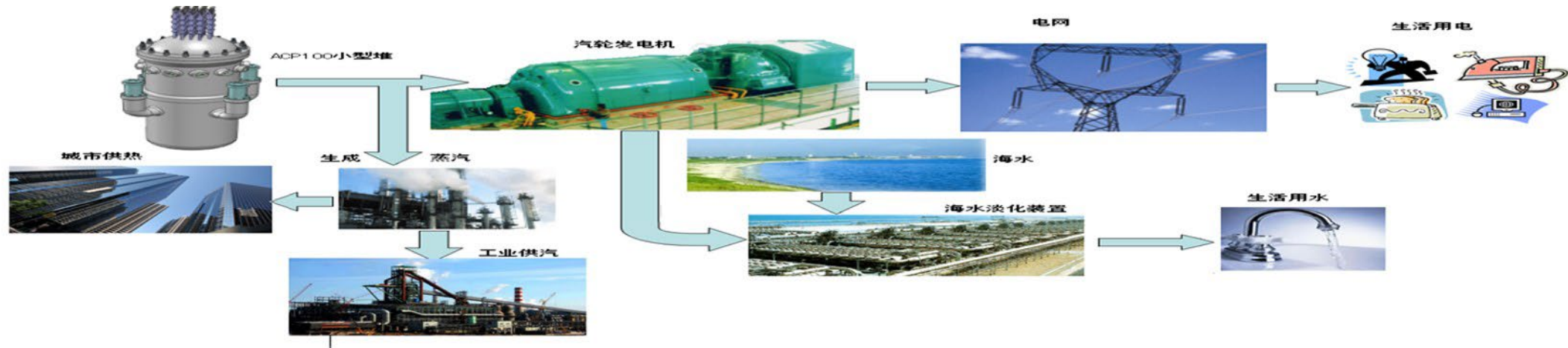
- (3) Nonelectrical applications for nuclear energy and the needs of developing countries for nuclear electricity
 - In 10 to 20 years, 70% of the energy consumption in developing countries will be nonelectrical in application, such as heat and transportation.

Background

- (4) Improving the economy by modular design and construction
 - Not like that of large nuclear power plants, the SMR achieves its economic by simplifying modular design and increasing the number of the modular.
- (5) Advantage for safety
 - SMRs with lower power and lower residual heating are suitable for passive safety facilities application.

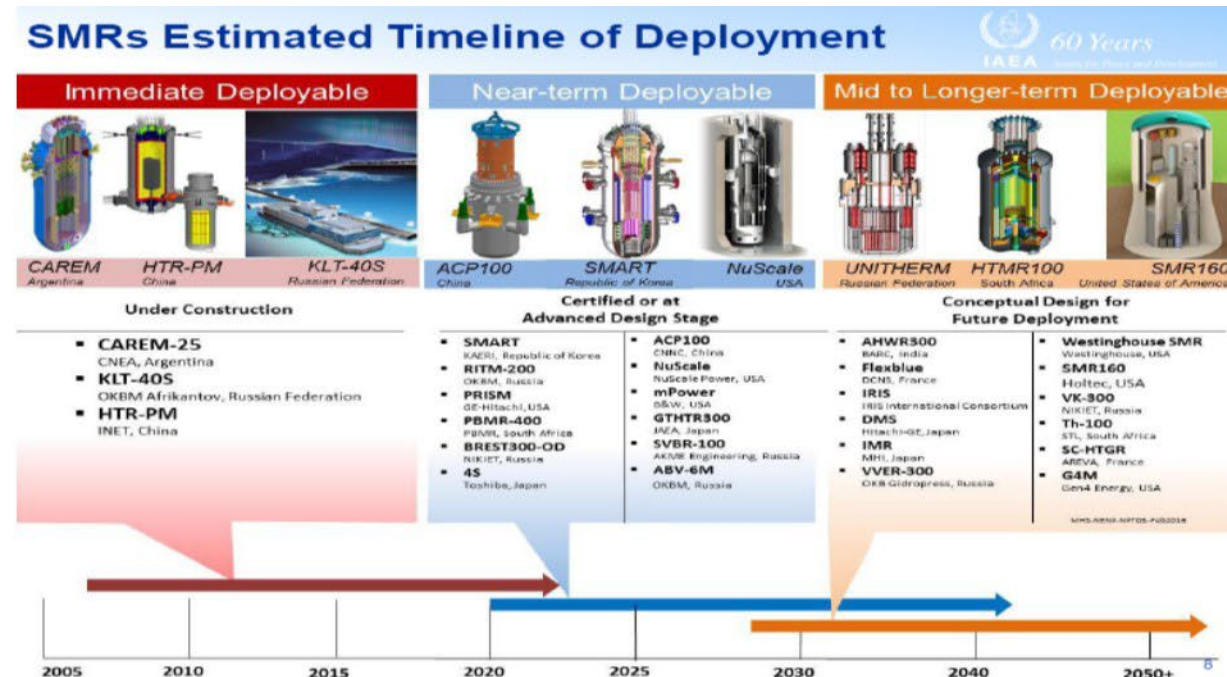
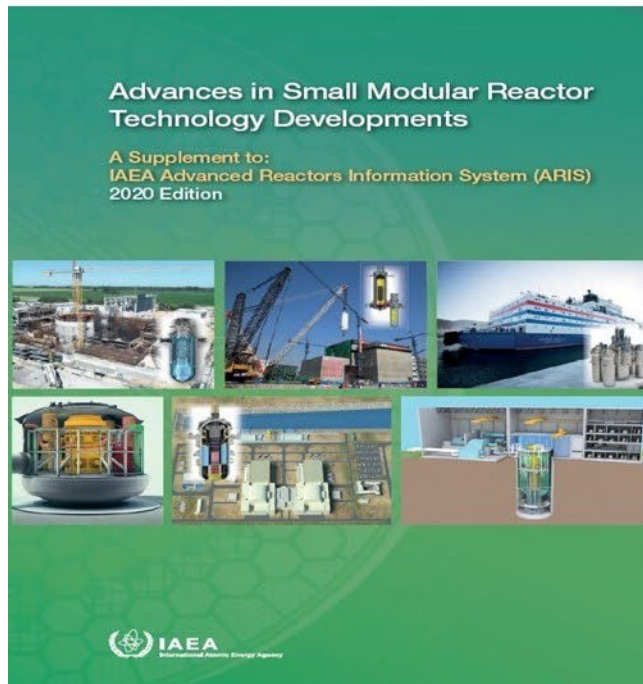
Background

- The SMR is suitable for a small electricity grid, district heating, process heating supply, seawater desalination. According to varying conditions, different countries have different goals.



Background

- Mainly developed and innovated SMR in different countries
- 2020 SMR book gives 72 reactors in 18 countries, **1/3 are PWR, and most of them are integrated reactor**

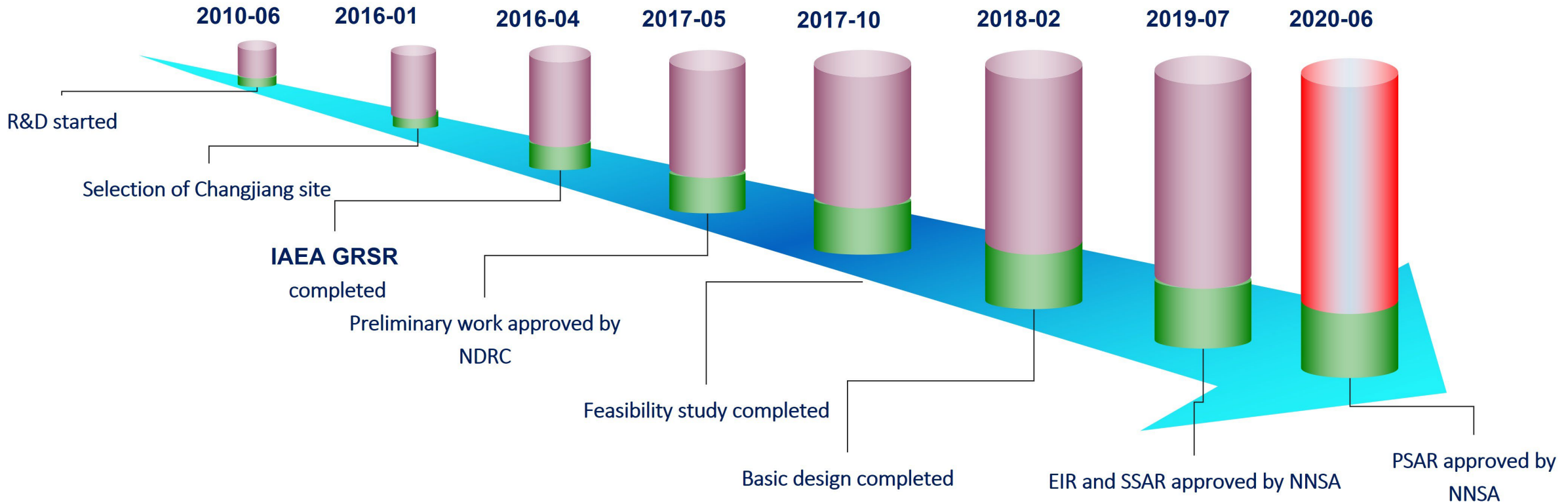


Introduction of ACP100

- CNNC SMR, named ACP100, is an innovative PWR based on existing PWR technology, adapting “passive” safety system and “integrated” reactor design technology
- CNNC started R&D on ACP100 in 2010
- The modular design technique is used to control the product quality and shorten the site construction period.

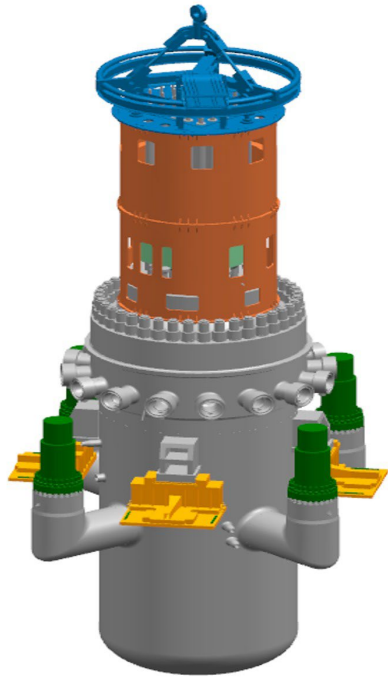
Introduction of ACP100

Roadmap of ACP100 development



Design parameters of ACP100

Main design parameters



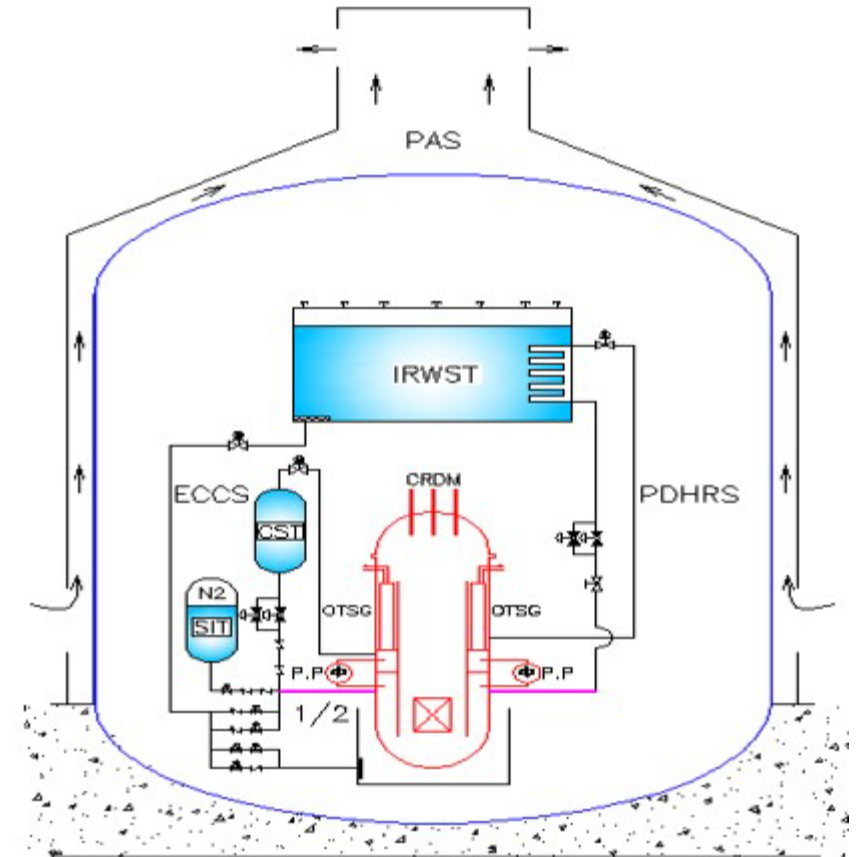
ACP100

<i>Thermal power</i>	385MWt
<i>Electrical power</i>	~125MWe
<i>Design life</i>	60 years
<i>Refueling period</i>	2 years
<i>Coolant inlet temperature</i>	282 °C
<i>Coolant outlet temperature</i>	323 °C
<i>Coolant average temperature</i>	303 °C
<i>Best estimate flow</i>	10000 m³/h
<i>Operation pressure</i>	15MPaa
<i>Fuel assembly type</i>	CF3 shortened assembly
<i>Fuel active section height</i>	2150 mm
<i>Fuel assembly number</i>	57

Design parameters of ACP100

Main design parameters

<i>Fuel enrichment</i>	4.45%
<i>Drive mechanism type</i>	Magnetism lifting
<i>Control rod number</i>	25
<i>Reactivity control method</i>	Control rod, solid burnable poison and boron
<i>Steam generator type</i>	OTSG
<i>Steam generator number</i>	16
<i>Main steam temperature</i>	>290 °C
<i>Main steam pressure</i>	4.5MPaa
<i>Main steam output</i>	560t/h
<i>Main feed water temperature</i>	105 °C
<i>Main pump type</i>	canned pump
<i>Main pump number</i>	4



Design parameters of ACP100

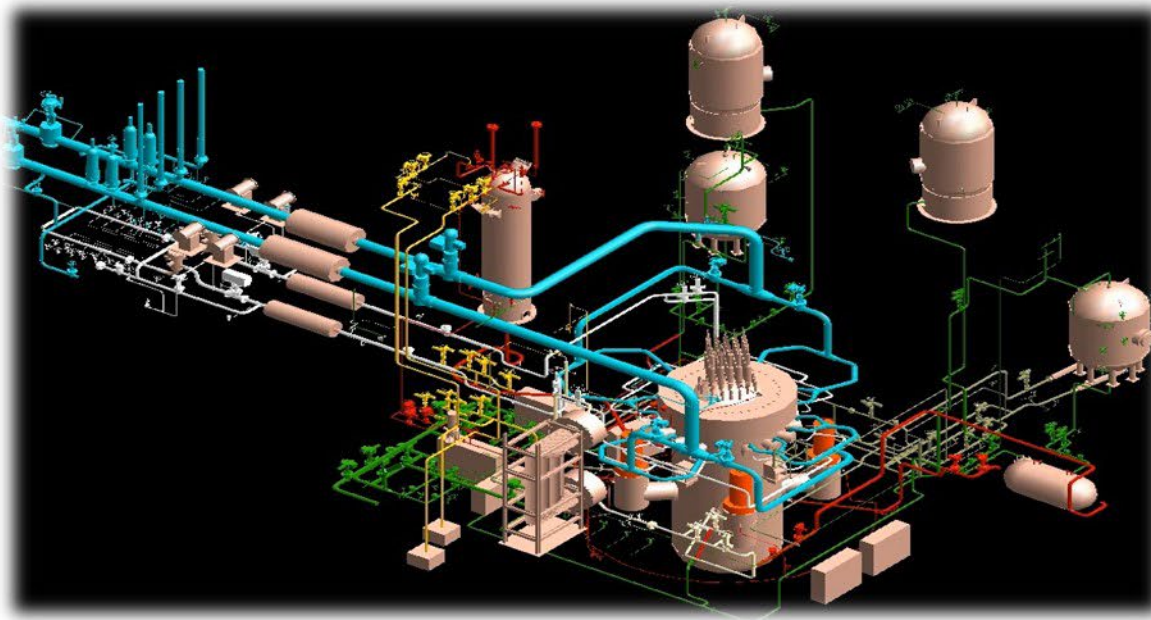
Main design parameters



<i>Reactor power-control operation program</i>	<i>primary constant average temperature</i>
<i>Thermal power plant operation model</i>	<i>Base load operation (Mode-A)</i>
<i>Plant design life</i>	<i>60 years</i>
<i>SSE level ground seismic peak acceleration</i>	<i>0.3g</i>
<i>Predicted Core Damage Frequency (CDF)</i>	<i><1E-7 Per reactor year</i>
<i>Predicted Large Release Frequency (LRF)</i>	<i><1E-8Per reactor year</i>

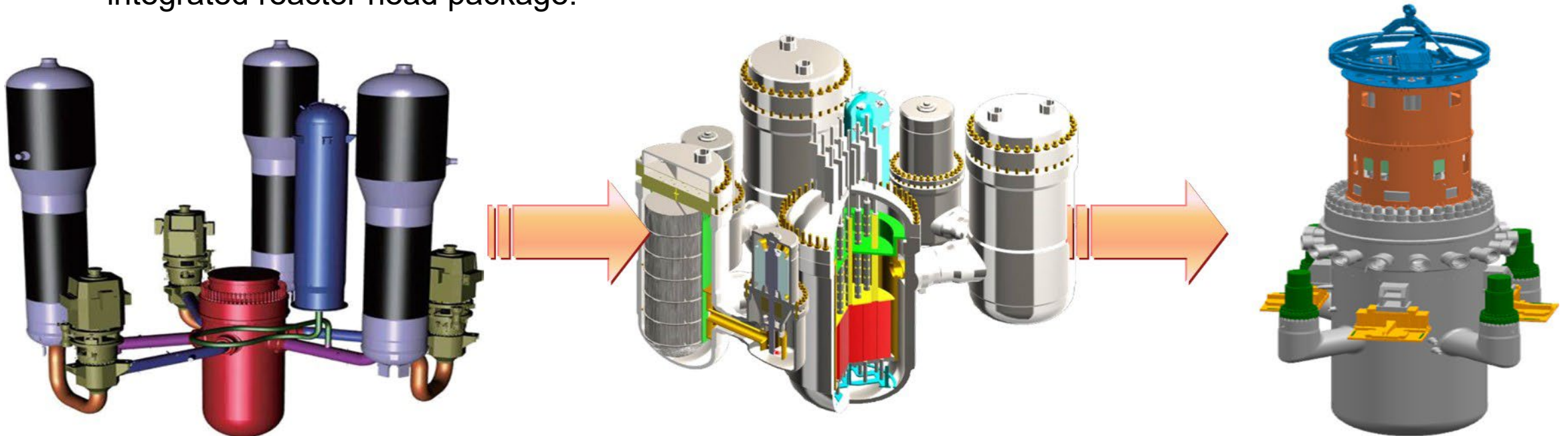
Design parameters of ACP100

- One reactor with one turbine



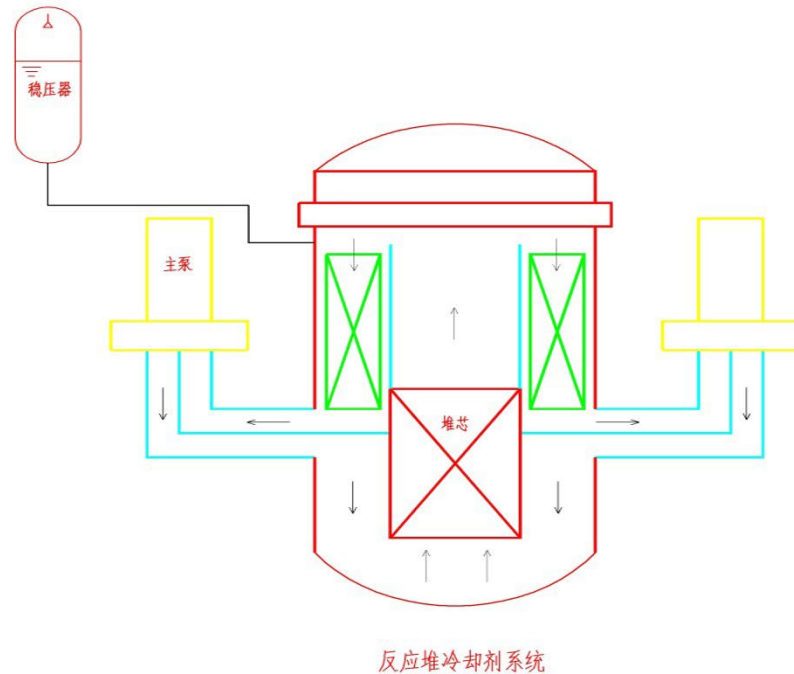
Technical Aspects

- Integral reactor module
 - The reactor coolant system has been integrated in the reactor module. The reactor module consists of a reactor vessel, once-through steam generators, canned motor pumps, reactor internals and an integrated reactor head package.



Technical Aspects

- (2) Reactor coolant system



- system function and composition

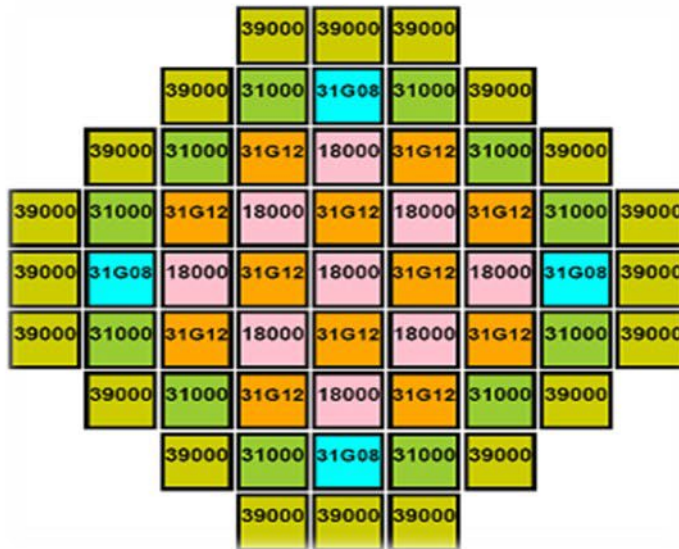
- 4 main pumps
- 16 OTSG
- 1 pressurizer

- system description

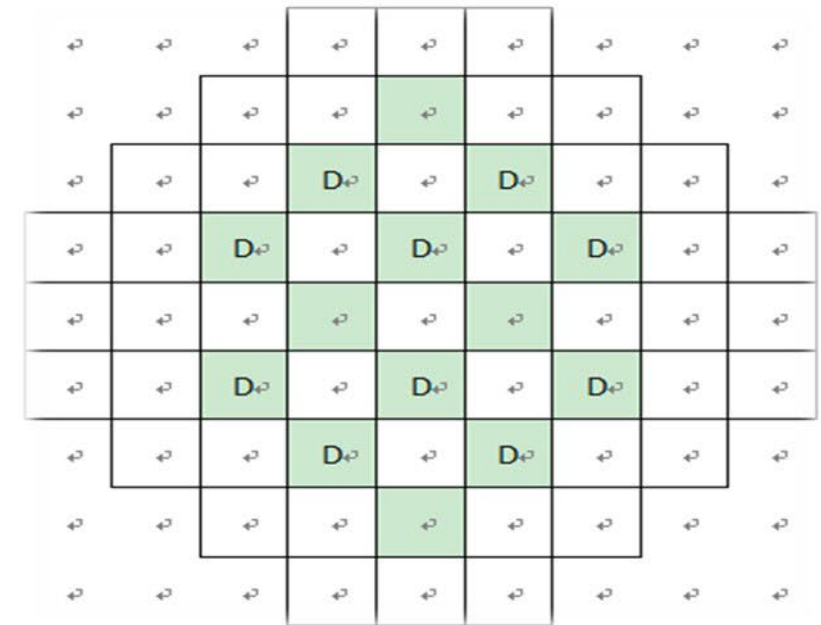
- operation pressure 15.0MPa
- core exit temperature 325°C

Technical Aspects

- (3) Reactor Core
 - 57 17X17 square fuel assembly with Gd₂O₃ solid burnable poison
 - Refueling period 24 months.



Core layout



Advanced core detection system

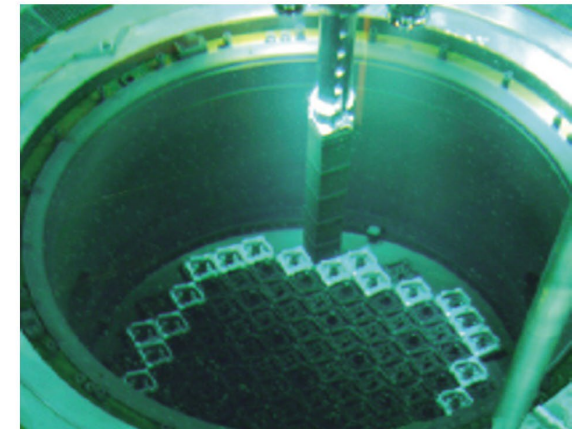
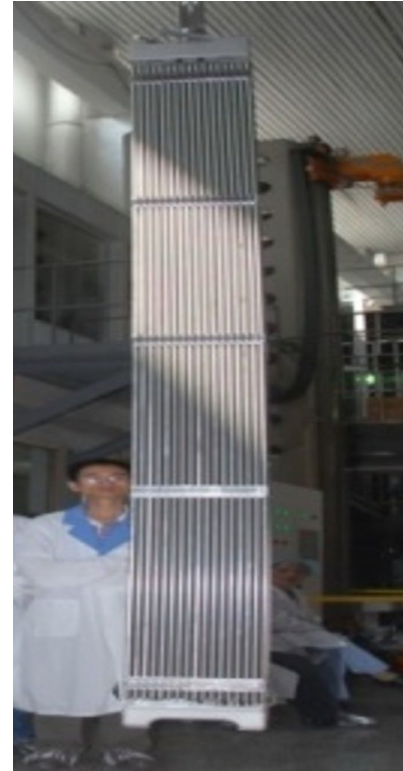
Technical Aspects

- (4) CRDM
 - step : 15.875mm/min
 - max travel speed : 72mm/min
 - electromechanical delay time : $\leq 150\text{ms}$
 - temperature : 200°C
 - pressure housing design life : 60 years
 - Min. cumulative step number of non-service : $6.0 \text{ E } 6$ steps



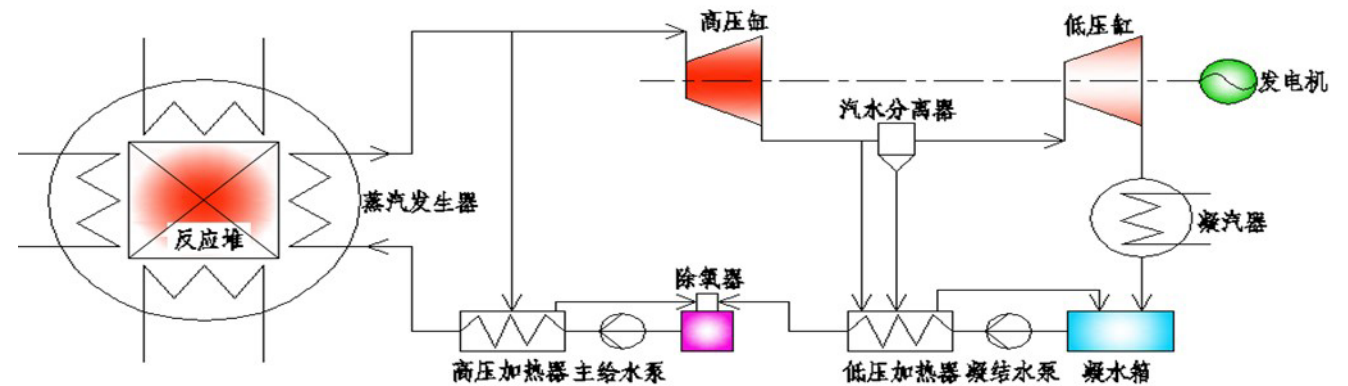
Technical Aspects

- (5) Fuel assembly
 - 17×17 square arrangement
 - Fuel rod : 264
 - guide tube : 24
 - instrumentation tube : 1
 - total height: ~2500mm
 - active length: 2150mm



Technical Aspects

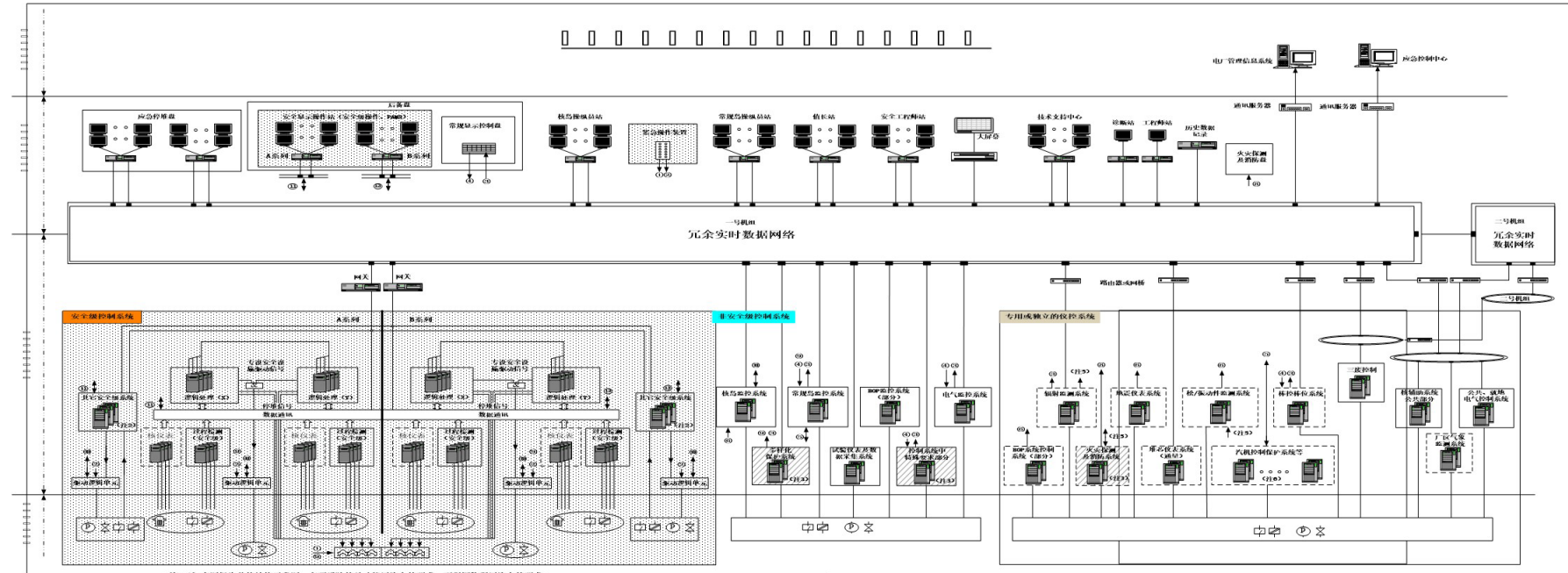
- (6) Main steam system
- system function and composition
 - main steam system
 - bypass system
 - moisture separator reheat system
- system description
 - operation pressure 4.5MPa
 - Temperature 285°C



Technical Aspects

- (7) I&C system
- Functionally divided into 4 layers :

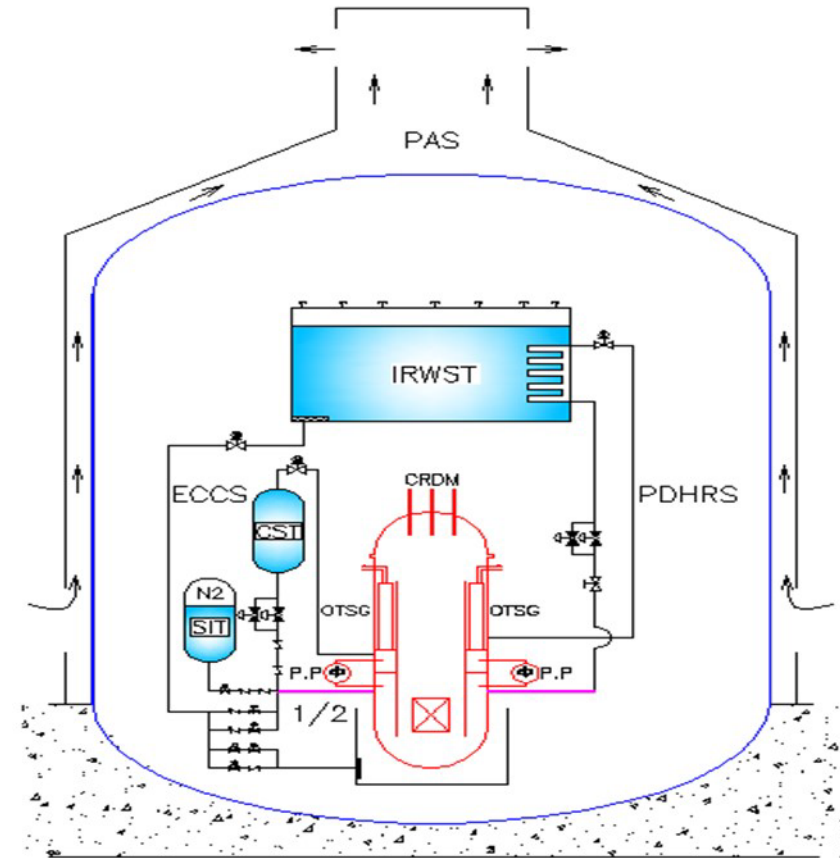
0. Interface layer
1. Auto control and covering layer
2. Operation and management information layer
3. Plant technical management layer



注：1-7 本图仅为总体结构示意图，主要控制的功能应满足下列要求：一个强物理控制的要求；
 2) 安全系统和安全系统相兼容，其中安全系统包括辅助工艺过程控制及支持系统安全部分、堆芯监测系统安全部分等；
 3) 非安全系统和安全系统相兼容，如专特化保护系统、火灾探测、消防、通风系统、及特殊工况下使用的系统和设备等；
 4) 非安全系统表示专用系统或相对独立的仪表控制系统；
 5) 非安全系统和安全系统相兼容，如堆芯监测系统与DCS在自动控制和保护时存数据接口；
 6) 非安全系统和安全系统相兼容，如堆芯监测系统与安全系统相兼容。

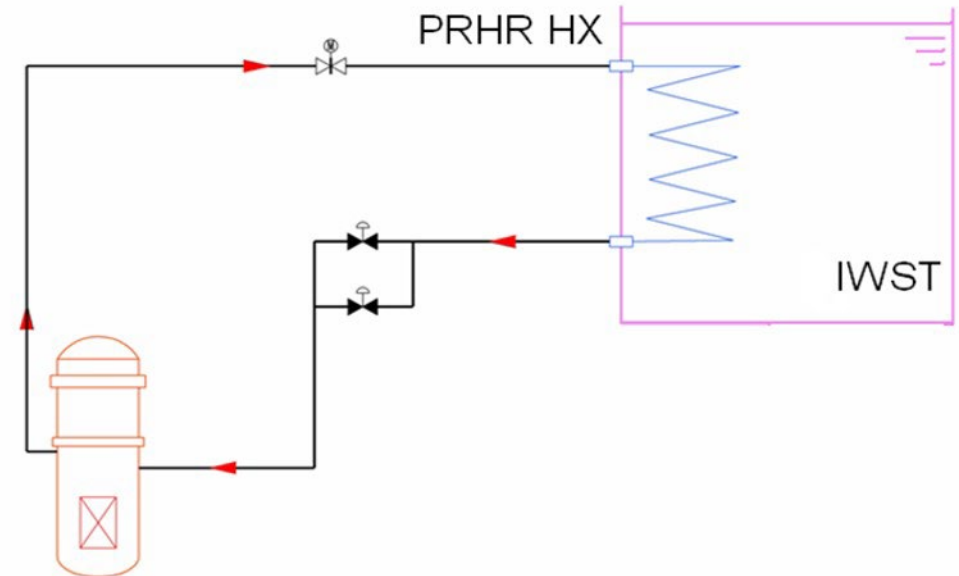
Technical Aspects

- (8) Fully passive safety system
- ACP100 adopts a fully passive safety system, which is illustrated in the figure:
 - passive core cooling system,
 - passive residual heat removal system
 - passive containment heat removal system
 - passive inhabitation system,
 - automatic depressurization system
 - passive hydrogen control system



Technical Aspects

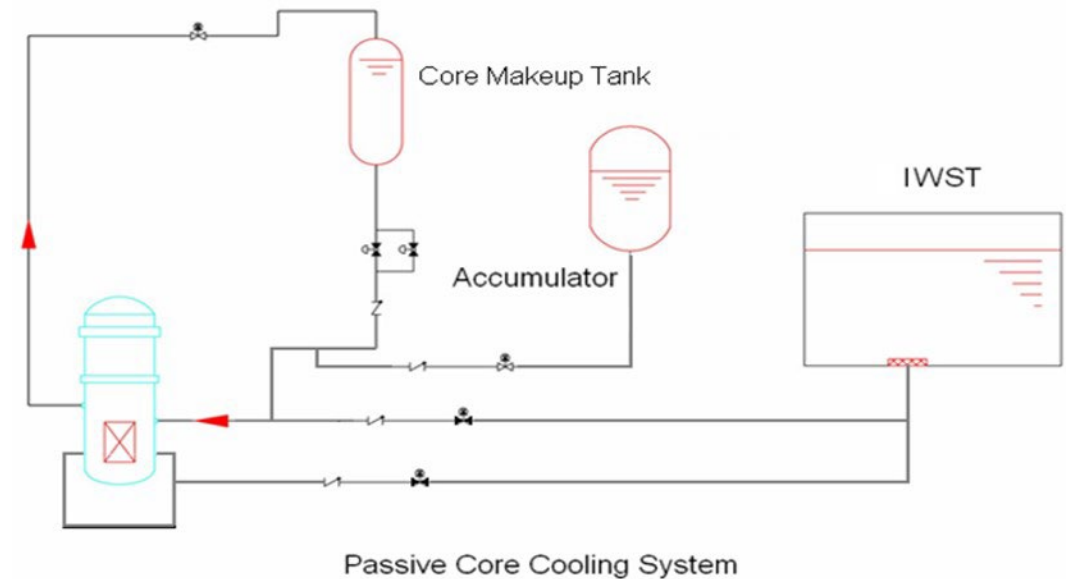
- (9) Passive residual heat removal system
 - The heat exchanger is mounted in the IWST. HX is usually filled with coolant.
 - Natural Circulation or if the RCPs are running, forced flow through the HX.
 - Flow through the HX-tubes from the RCS transfers heat to the IWST contents.



Passive Residual Heat Removal system

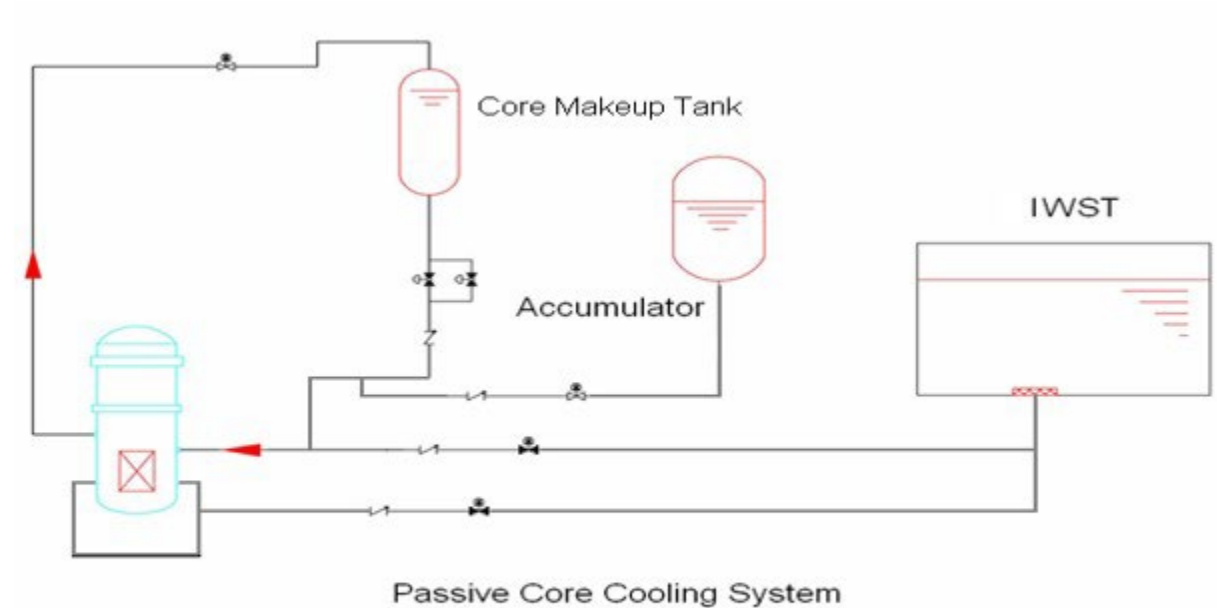
Technical Aspects

- (10) Passive core cooling system
 - Provides the Reactor coolant system emergency makeup.
 - Safety injection: Provide adequate core cooling for the SBLOCA (i.e., SGTR) by:
 - CMTs
 - Accumulators
 - IWST
 - After CMTs, Accumulators, & IWST have injected, containment is flooded sufficiently to provide recirculation flow.



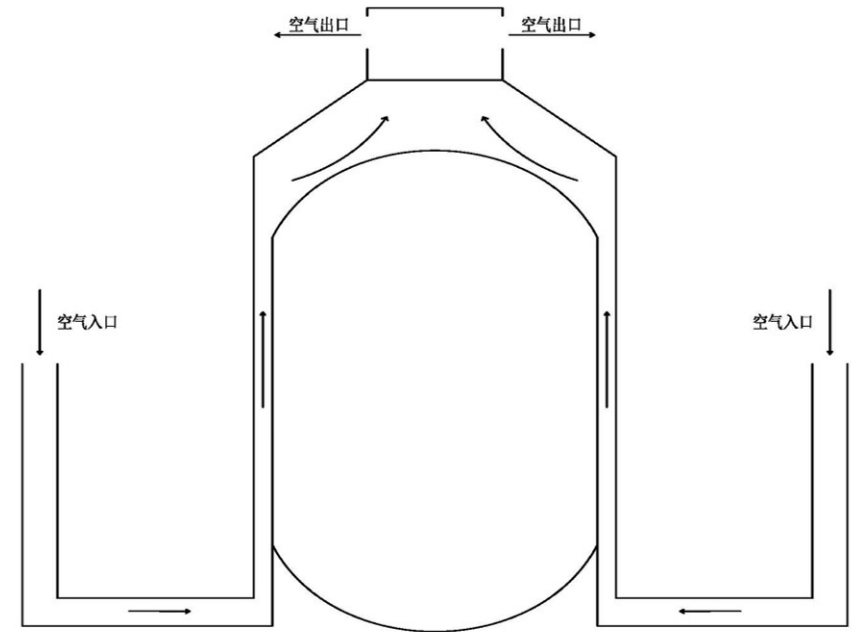
Technical Aspects

- (11) Passive Reactor Cavity Flooding System
 - Provides a means of external reactor vessel cooling under assumed severe accident condition.
 - Prevents the core from melting



Technical Aspects

- (12) Passive containment cooling system
 - Provides long-term heat removal from the containment in the case of any DBA & BDBA, including those associated with blackout and spray system failure; Steam condenses on containment HX.
 - Condensate collects in IWST/Sump via gutter arrangement.
 - Core heat is ultimately transferred through the containment HX into the surrounding atmosphere by natural circulation.



Technical Aspects

- (13) Other safety systems
 - Provide automatic depressurization (ADS) in the event of a SBLOCA
 - Passive combustible gas control in containment

Safety and licensing strategy

- Codes and Standards applied by ACP100
 - Level 1, Laws - Issued by the Congress (mandatory).
 - Level 2, Codes and Regulations - Issued by the State Council (mandatory). Setting up administrative scope, principles, organizations and its functions etc ;
 - Level 3, Departmental Rules - Issued by governmental organizations (mandatory). Defining the implemental methods based on the Regulations. Setting up nuclear safety objectives and basic requirements ; (NNSA)
 - Level 4, the Guides - Issued by the Governmental organizations (recommended). Recommending the methods or procedures to satisfy the safety requirements ;
 - Level 5, Technical documents - Issued by the Governmental organizations (referential).

Safety and licensing strategy

Codes and Standards applied by ACP100



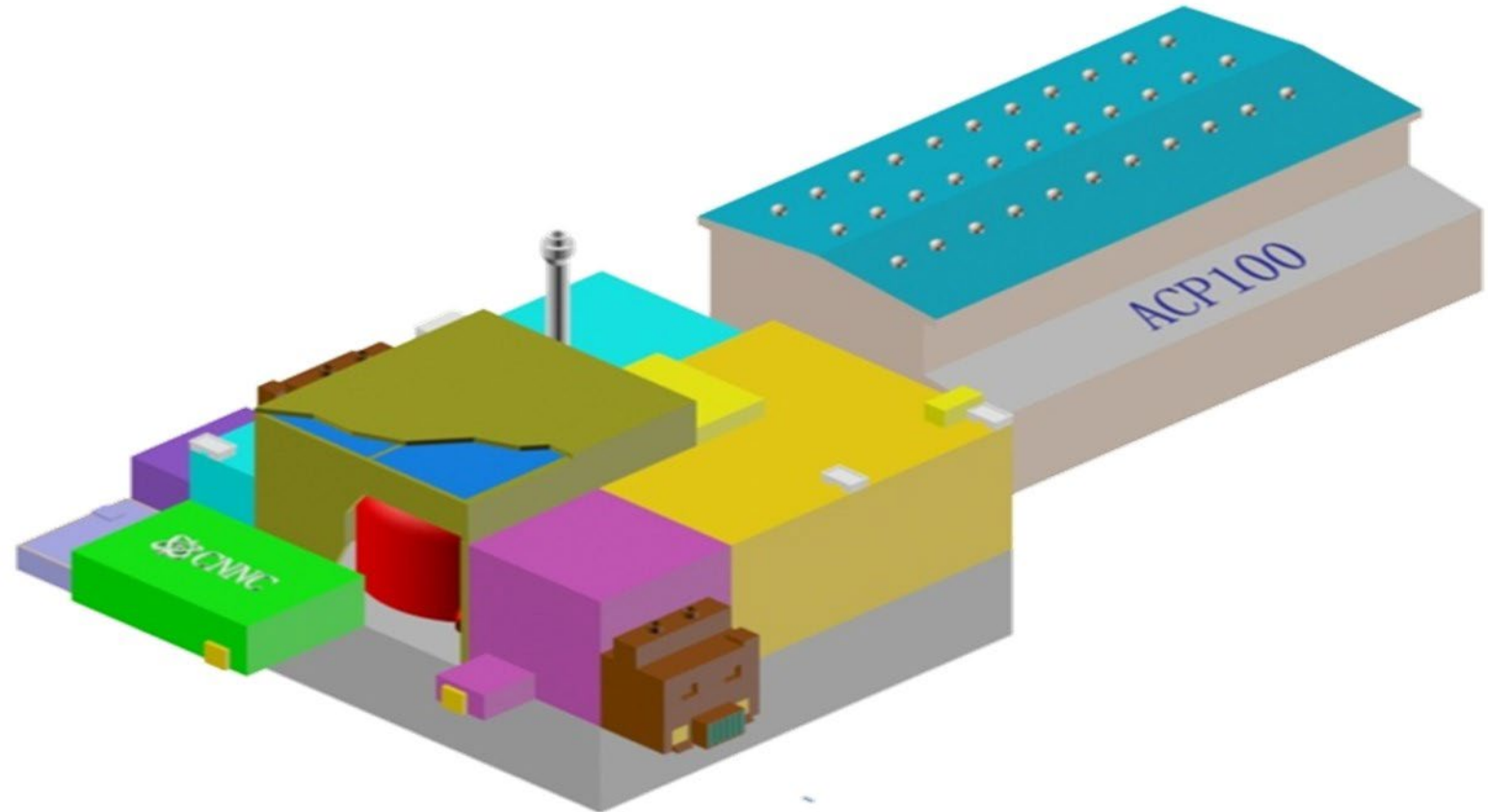
Safety and licensing strategy

- ACP100 Safety design conception
 - No active Emergency Core Cooling System
 - No active containment spray and recirculation system.
 - No need for operator intervention after accident for 72 hours.
 - No safety-related emergency AC power.
 - NSSS integral design minimizes both the probability and impact of design basic accident (DBA).
 - Mitigate DBA without non-safety system. Emergency planning zone is limited inside the site boundary.

Safety and licensing strategy

Safety design in defense-in-depth

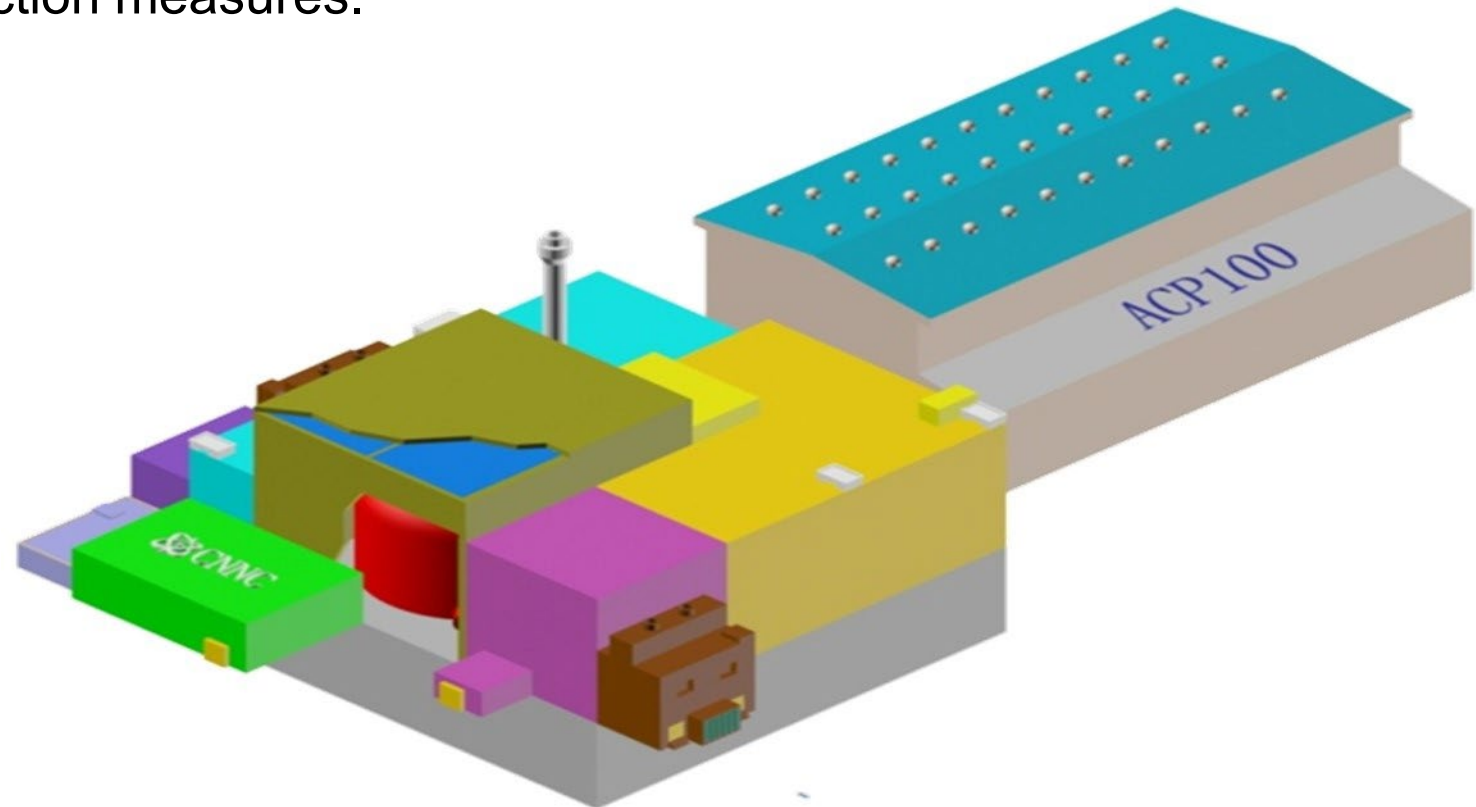
- Defense in depth (DiD) is critical element of safety principles and has been incorporated into all the safety related activities to guarantee that those activities are protected under overlapped safety measures. 5 layers DiD measures are incorporated into ACP100 design



Safety and licensing strategy

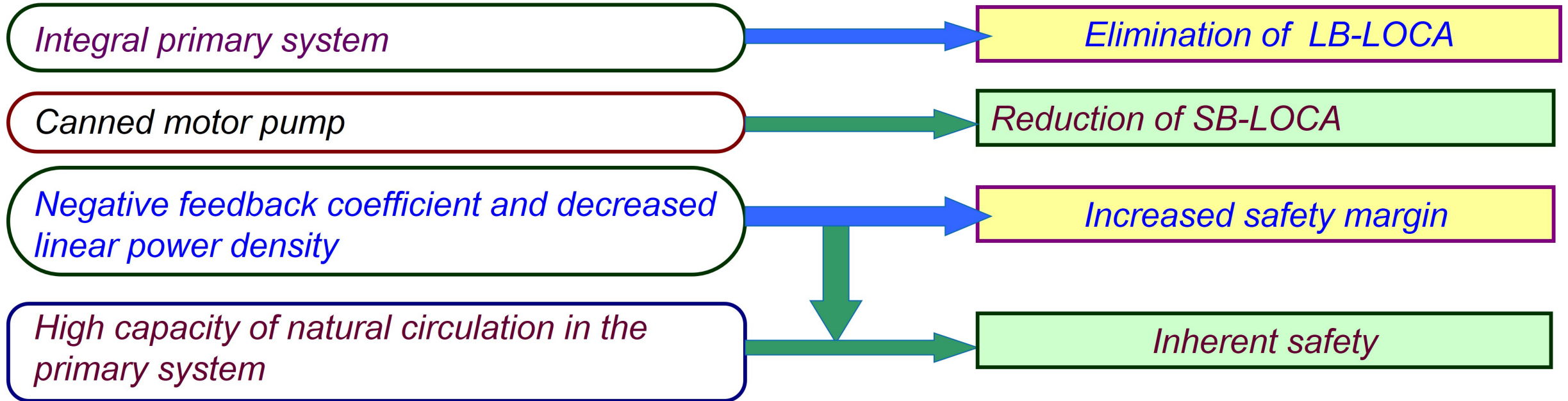
Safety design in defense-in-depth

- ACP100 has robust multiple physical barriers to prevent radioactive materials release. Those barriers are composed of fuel matrix, cladding, reactor coolant system barrier, containment and aircraft protective containment. ACP100 designs guarantee the effectiveness of every barrier and provide protection measures.



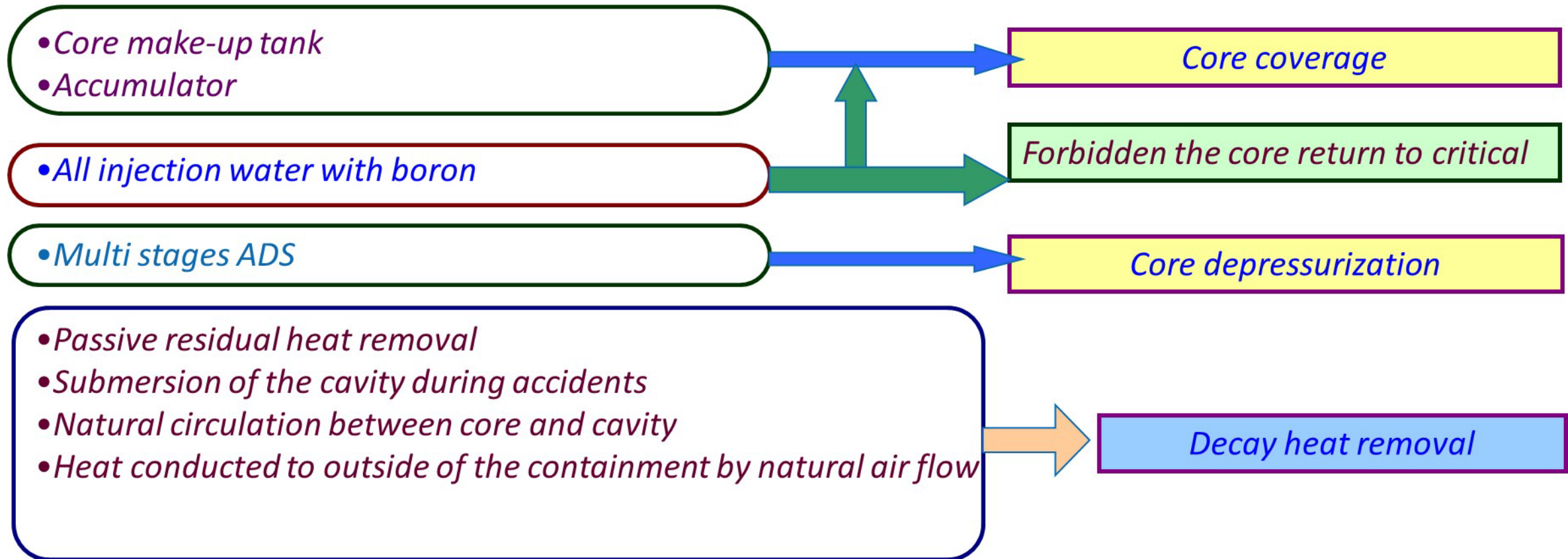
Safety and licensing strategy

Special design aspects



Safety and licensing strategy

Passive safety system

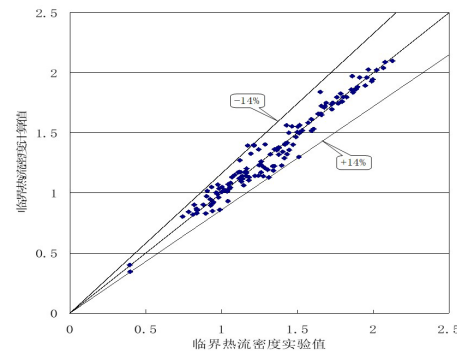
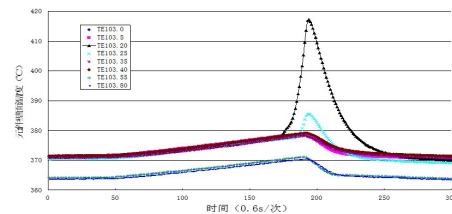


Safety and licensing strategy

Deterministic and Possibility Safety analysis

Deterministic Safety analysis

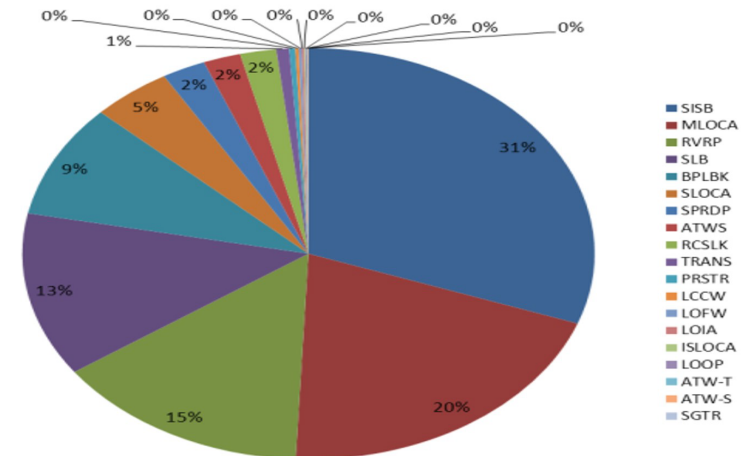
- 7 categories including 50 kinds of incidents and accidents
- 15% of thermal margin achieved



Possibility Safety Analysis (PSA)

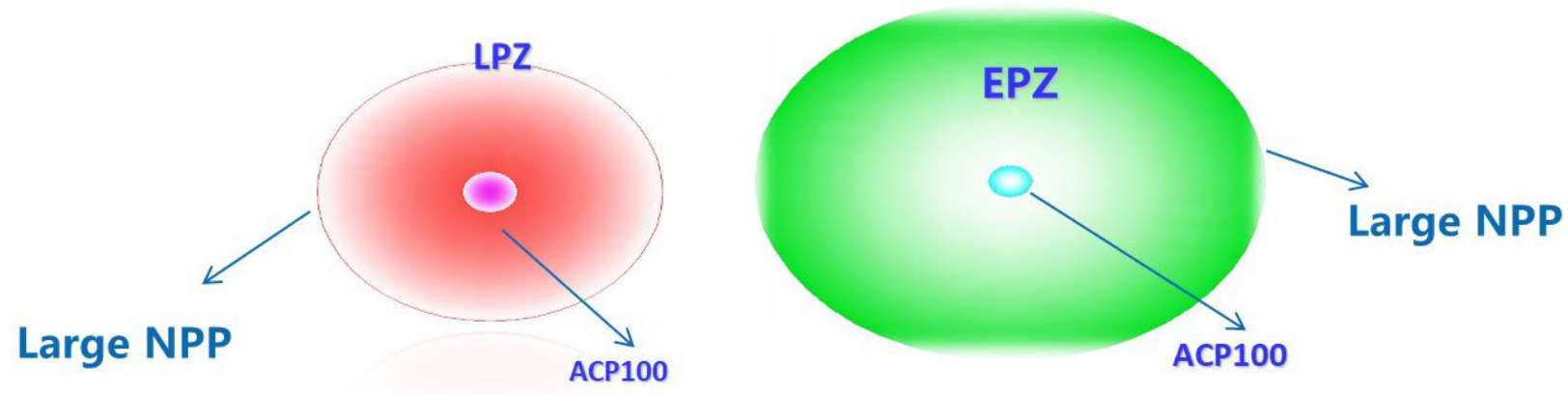
CDF: 1.91×10^{-7}

LOF: 10^{-8} level



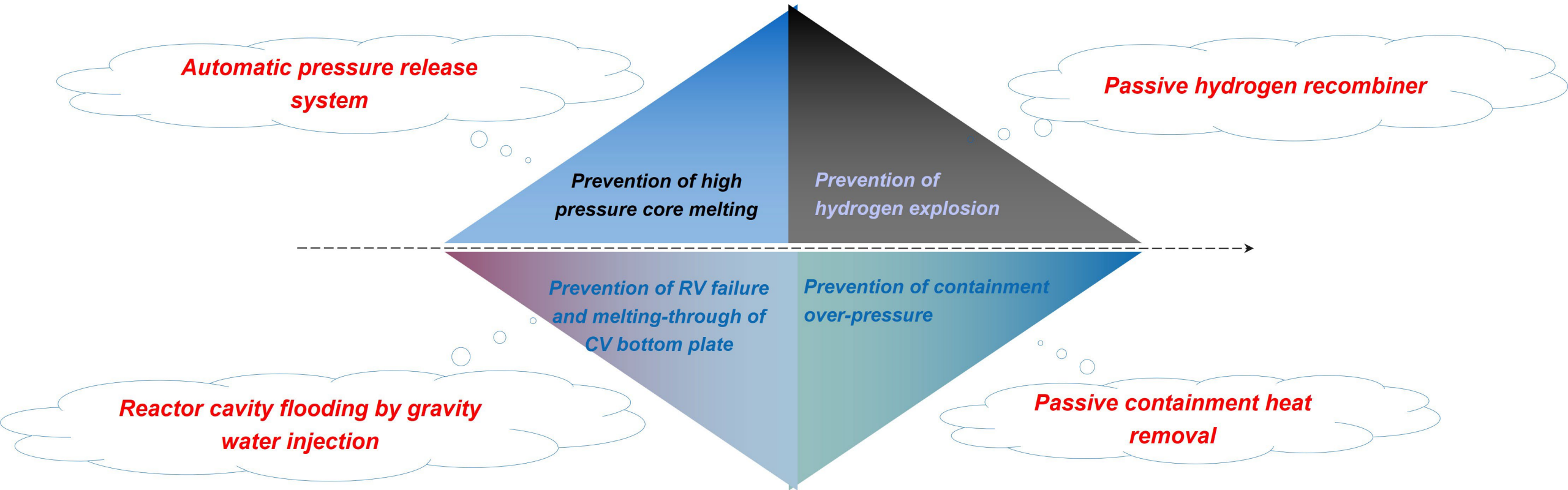
Safety and licensing strategy

- Non-residential Area and Planned Restricted Zone Study
 - Non-residential area (EAB): Less than 300 m; (for large reactor 500m)
 - Planned restricted zone (LPZ): Less than 500 m; (for large reactor 5km)
 - Emergency plan zone (EPZ): Internal zone Less than 500 m; (for large reactor 3~5 km) External zone Less than 600 m. (for large reactor 7~10 km)



Safety and licensing strategy

Severe accident prevention and Mitigation measures



Safety and licensing strategy

- Third party verification

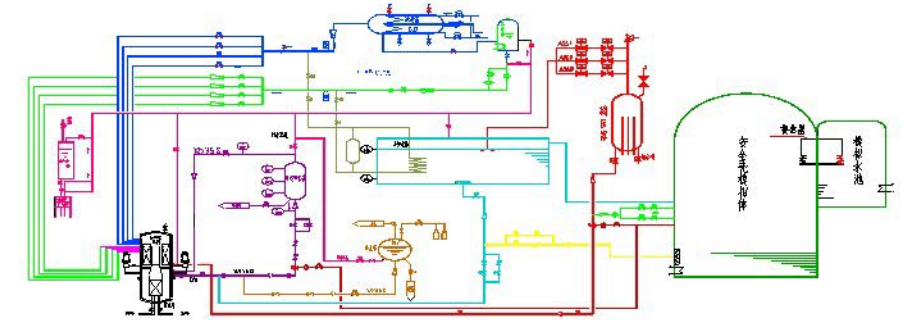
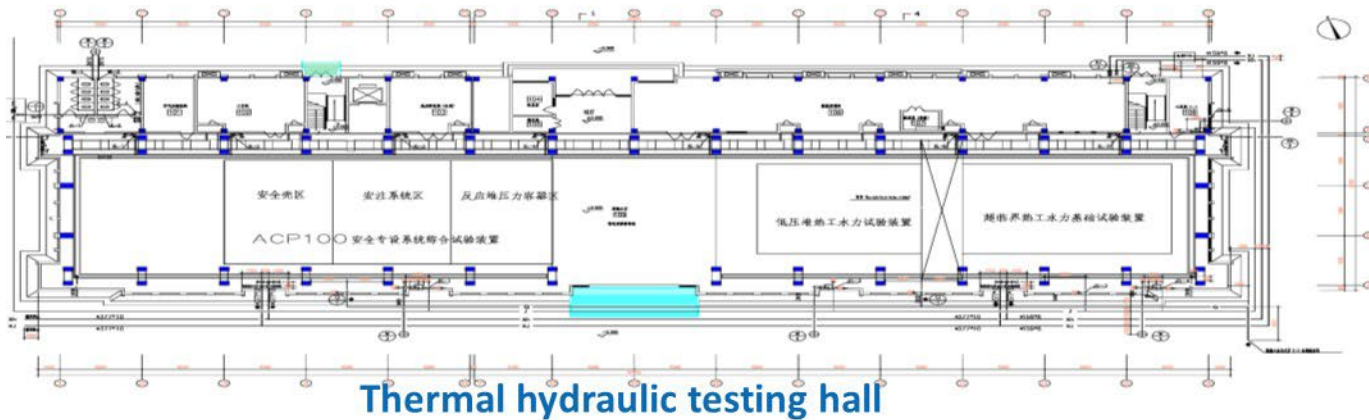
- IAEA gave the review comments on ACP100 Generic Reactor Safety Review GRSR report on April 22, 2016, the 1st SMR completion of GRSR in the world.

“According to the safety documentation, the ACP100 plant is an innovative design that belongs to the SMR class of NPPs and deploys passive safety features. It can be expected from new designs that they are capable of dealing with extreme environmental conditions and multiple failures to assure that early or large radioactive releases are practically eliminated.”



Testing & Verification

- Seven test research
 - Control rod drive line cold and hot test
 - Control rod drive line anti-earthquake test
 - Internals vibration test research
 - Fuel assembly critical heat flux test research
 - Passive emergency core cooling system integration test
 - CMT and passive residual heat removal system test research
 - Passive containment heat removal testing



Passive emergency core cooling system

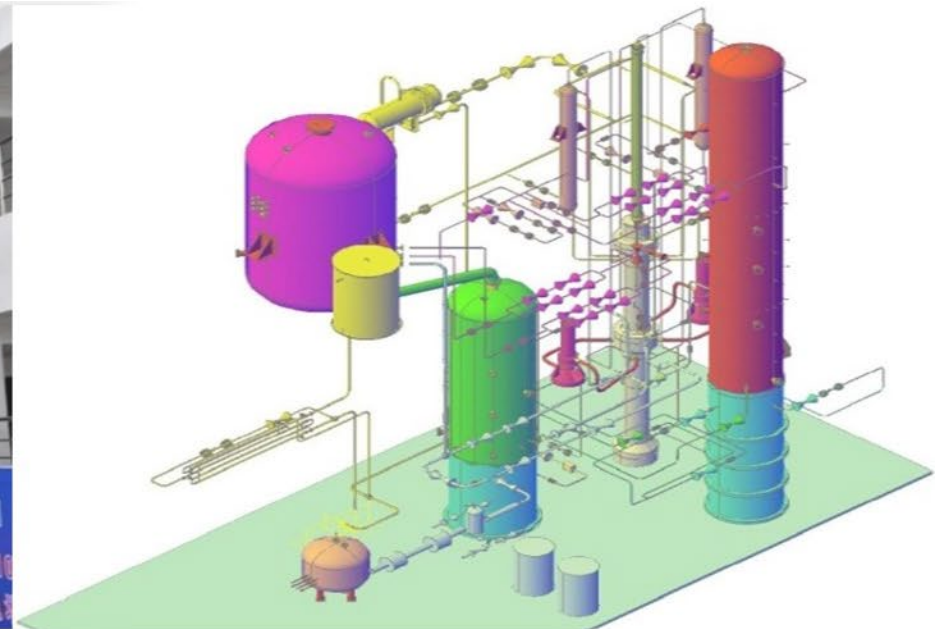
Testing & Verification

- Seven test research
 - Control rod drive line cold and hot test
 - Control rod drive line anti-earthquake test



Testing & Verification

- Seven test research
 - Passive emergency core cooling system integration test
 - Over 3 years, CNNC has constructed the most comprehensive passive engineering safety system testing facility. Core cooling system integration testing, Passive residual heat removal system testing had finished on this facility



Testing & Verification

Seven test research

- Fuel assembly critical heat flux test research



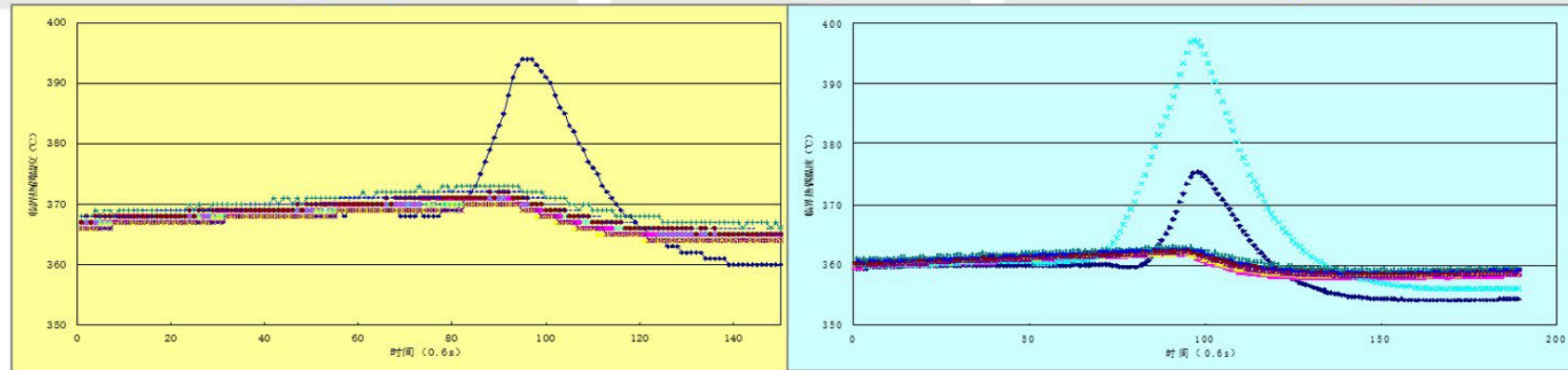
CHF testing facility



CHF testing tube

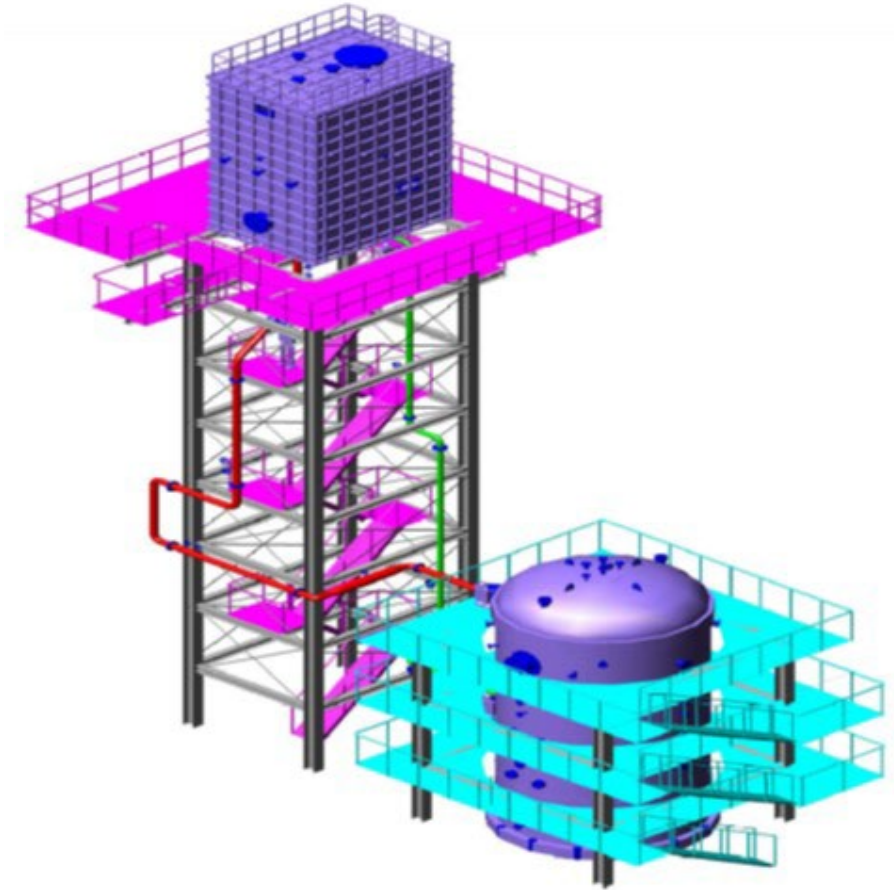


CHF heating assembly



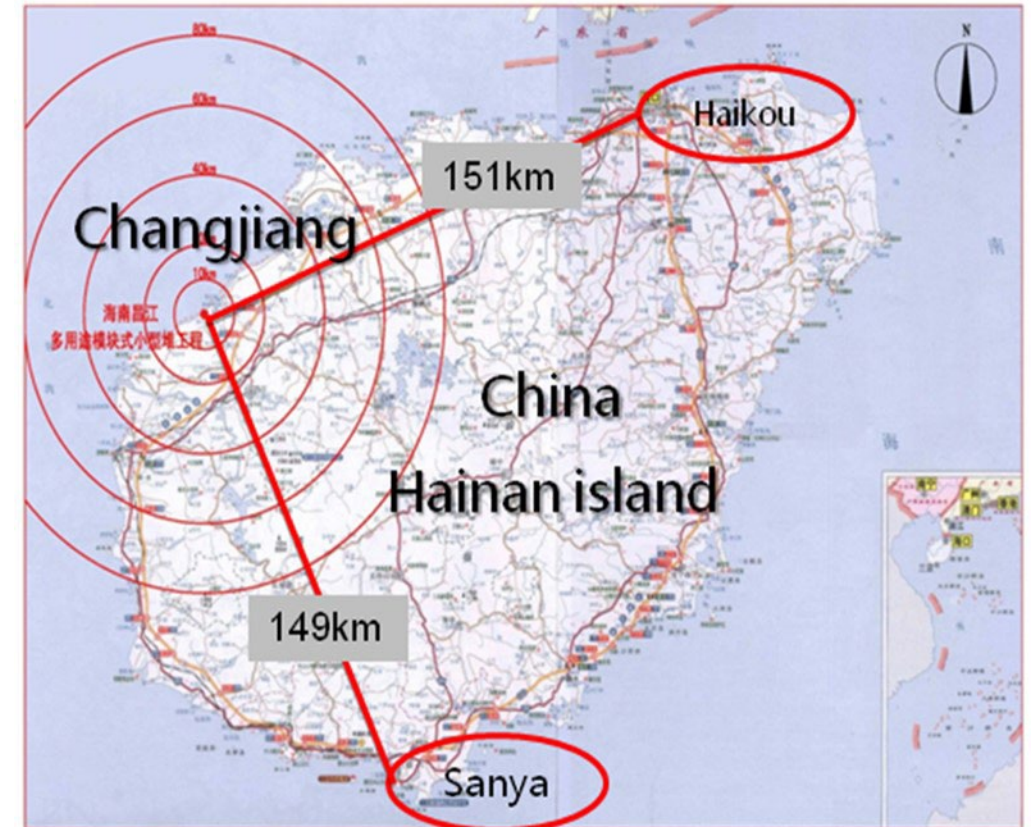
Testing & Verification

- Seven test research
- Passive containment heat removal testing
 - The results of the testing indicate the passive containment heat removal system is sufficient to conduct the heat to the ultimate heat sink

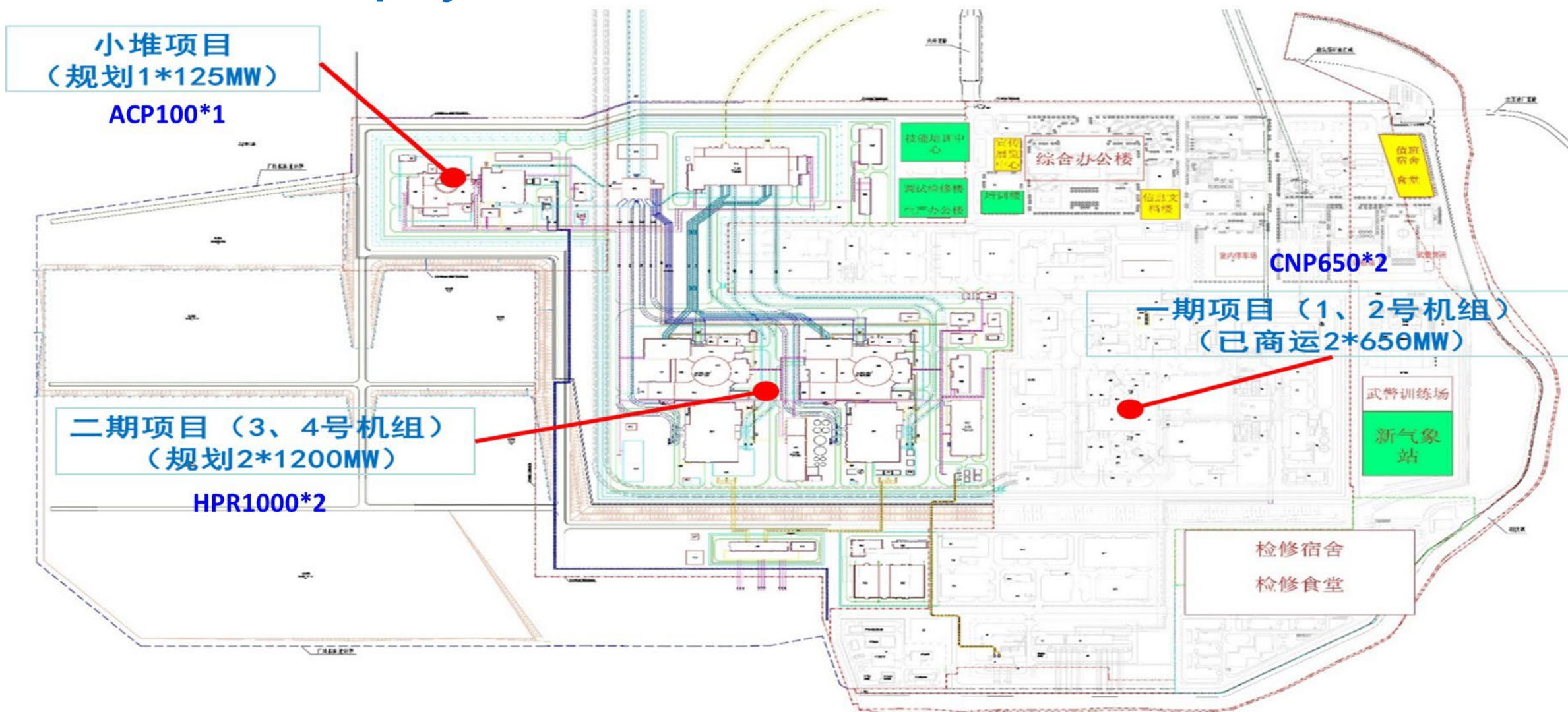


Demonstration project of ACP100

- Changjiang nuclear power site, Hainan, China, as illustrated in Figure, was chosen to build the first of a kind (FOAK) ACP100 demonstration project.
- **FCD in July, 2021.**
- Construction period of FOAK 55 months, target **commercial operation in 2026**



Demonstration project of ACP100



Demonstration project of ACP100

- Major equipment, such as the Reactor Pressure Vessel, steam generator and turbine generator are already in the manufacturing stage.



RPV主泵接管锻件



RPV支撑段筒体堆焊



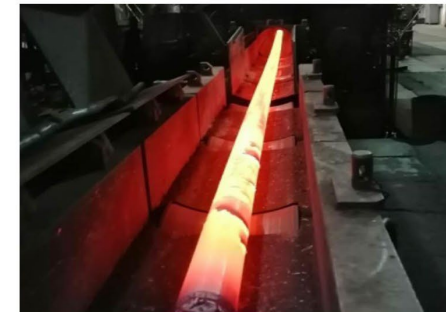
RPV容器法兰堆焊



RPV主泵接管待堆焊



主泵试验回路



SG钛管热轧



主泵电机组装

Demonstration project of ACP100

- Site preparation on 18 July 2019



小堆施工现场



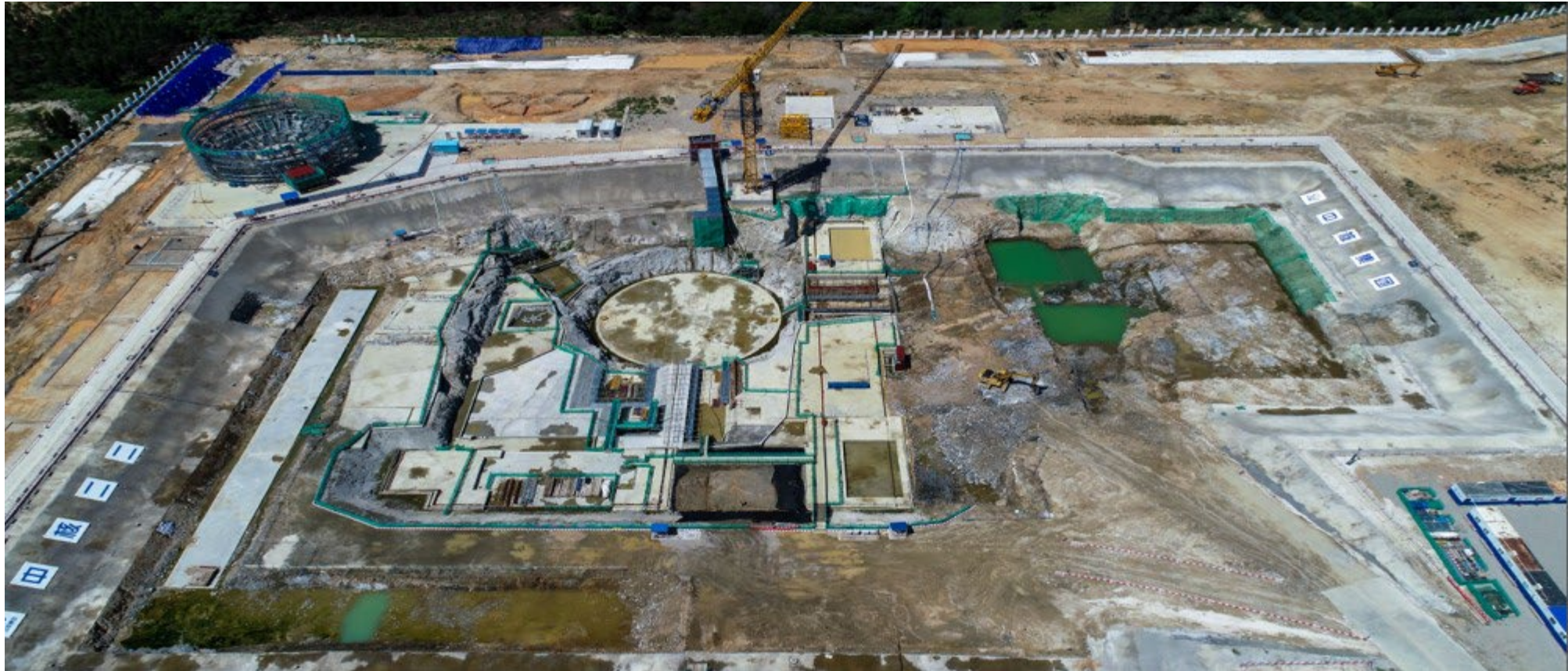
Demonstration project of ACP100

- Site preparation on 31 December 2019



Demonstration project of ACP100

- Site preparation on 30 June 2020



Demonstration project of ACP100

- Site preparation on 26 February 2021



Demonstration project of ACP100

- FCD on 13 July 2021

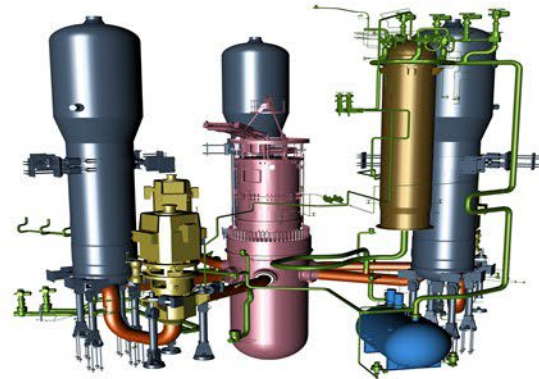
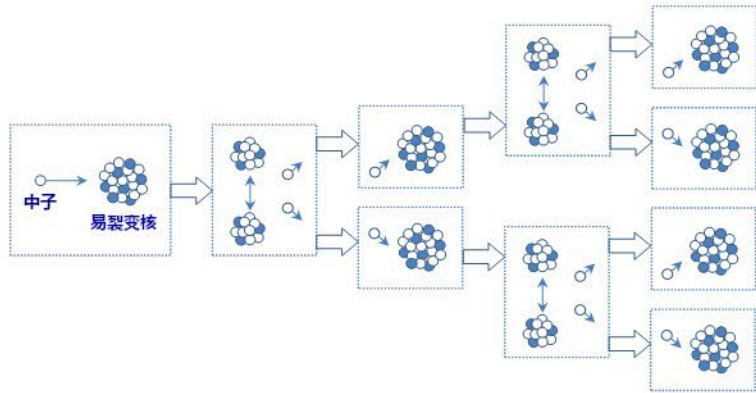


Multiple applications of ACP100

- Merit of nuclear energy :
 - Higher density, lower carbon emission
 - Stable operation, no fluctuation

1kgU²³⁵=2700 Ton Coal

1000MWe NPP de-carbonize 3 million Ton Coal per year



Reactor



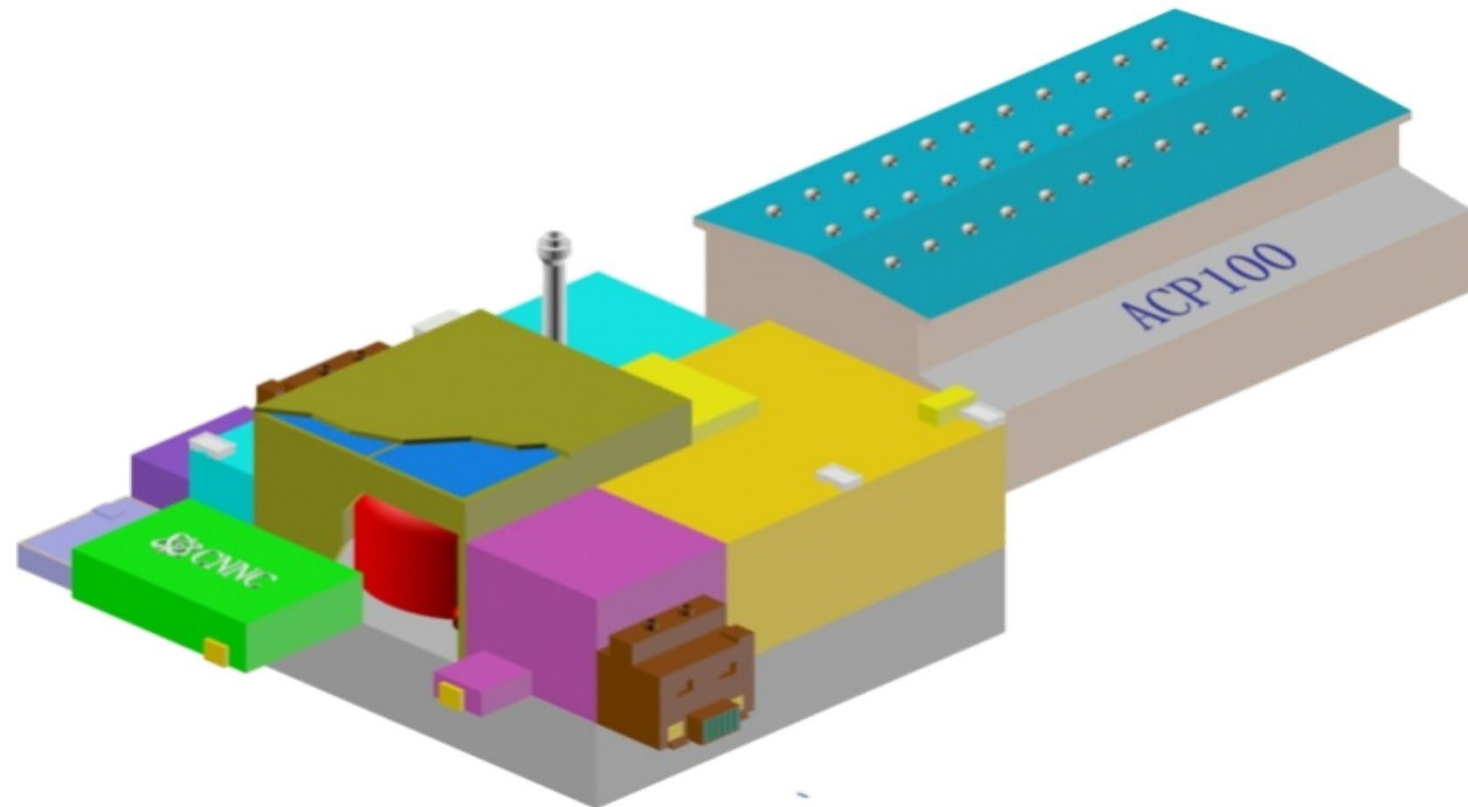
Multiple applications of ACP100

- 80% end user of energy is used for electricity, heating and transportation
- In year 2019, energy consumption of China (electricity 23.9%, heating 45% Industry 24%, civil 21%), transportation 11%)

ENERGY	Electricity	Heating		Transportation	Others
		Industry	Civil		
	23.9%	24%	21%	11%	--

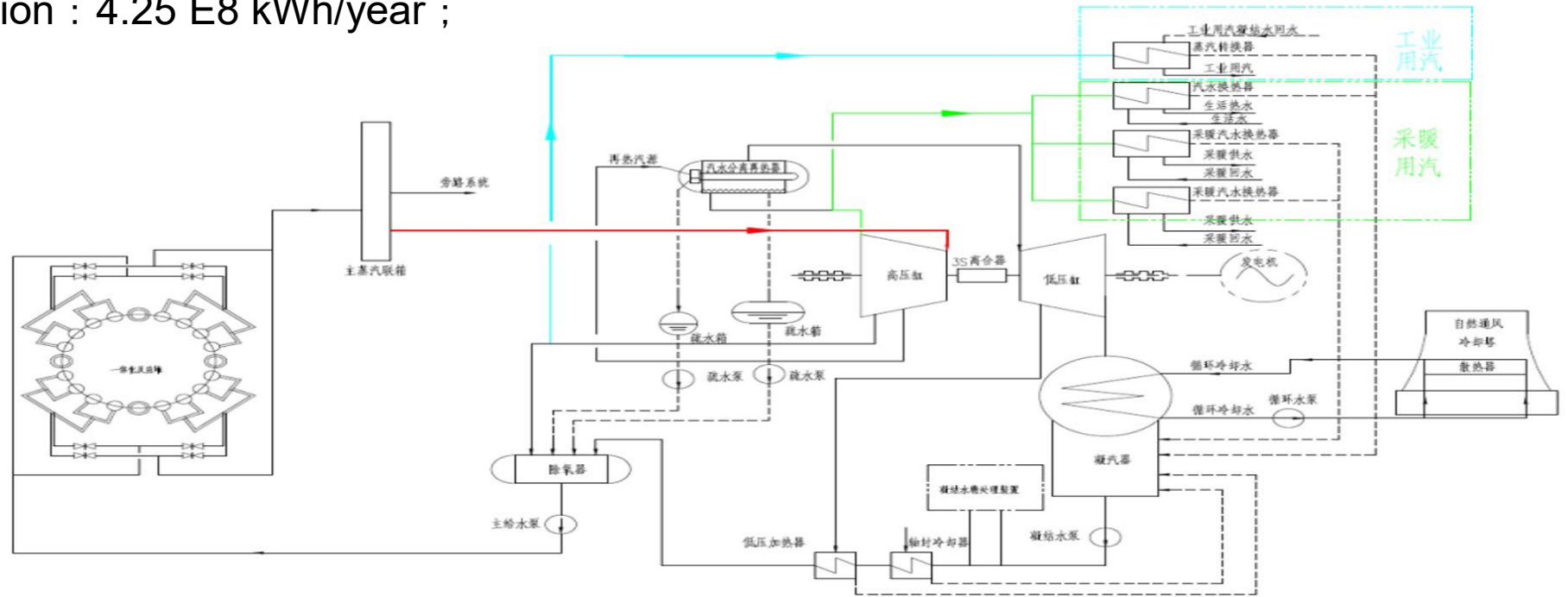
Multiple applications of ACP100

- In the fields of Electricity Generation
 - Hainan Changjiag Project (under construction) :
 - Electrical Power 126.5Mwe, Refueling period 24 months. Electrical generation 109 Kwh/year, Satisfy for half million families.



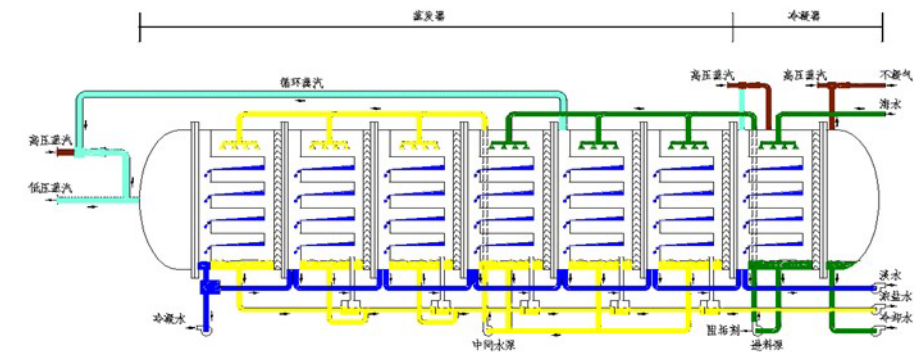
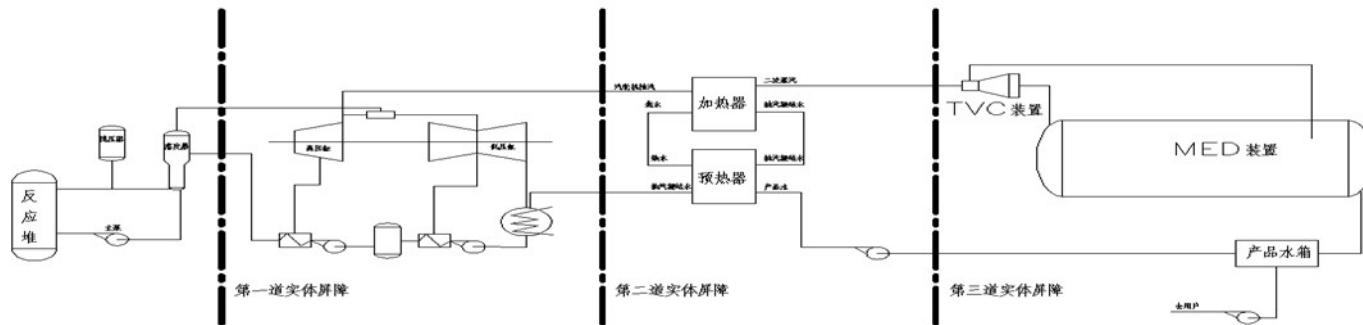
Multiple applications of ACP100

- In the fields of District heating & Electricity Generation
- Gansu project (feasibility study)
 - Thermal generation : 6.55 million GJ/year ;
 - Electrical power : 80 MWe ;
 - Electrical generation : 4.25 E8 kWh/year ;



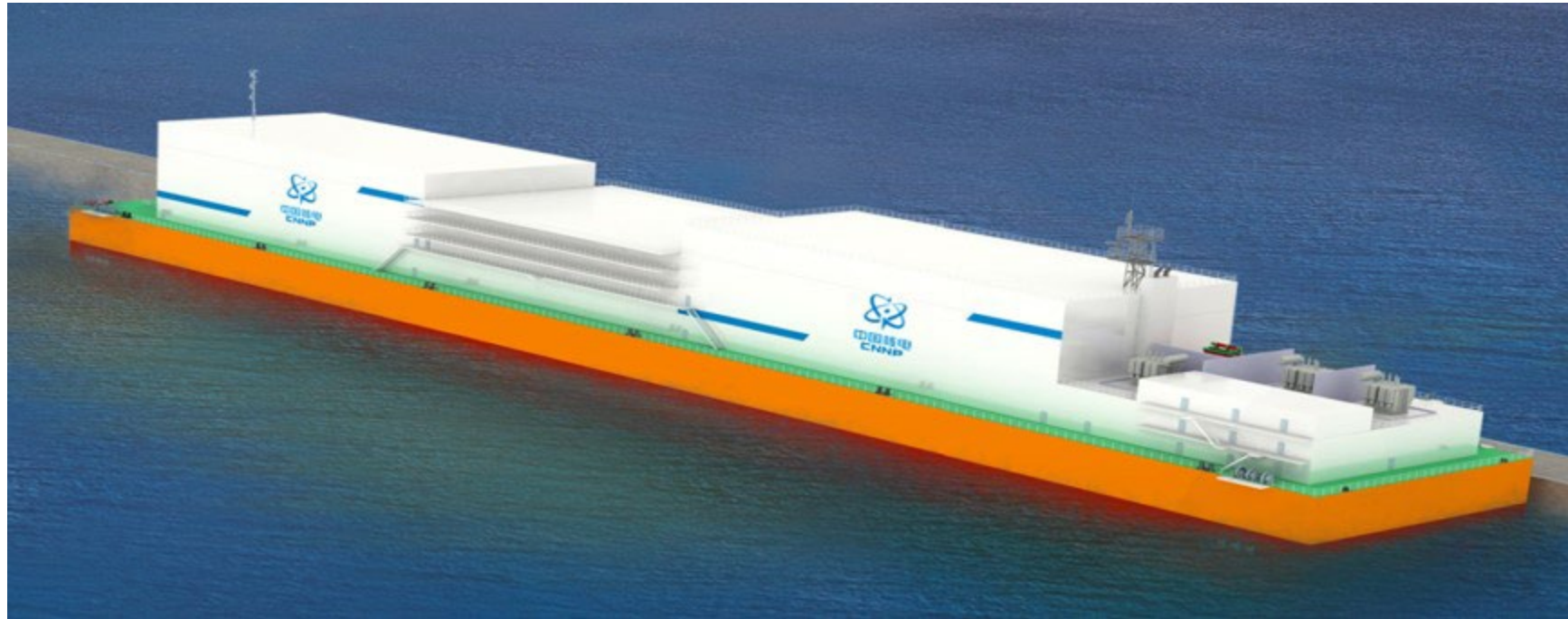
Multiple applications of ACP100

- In the fields of Seawater Desalination & Electricity Generation
 - Fujian Putian project (feasibility study)
 - (Low Temperature Multi-effect Sea Water Desalination Facility):
 - Fresh water generation : 48,000 m³/day
 - Electrical generation : 75 MWe



Multiple applications of ACP100

- In the fields of Floating Nuclear Power Plant
 - Shandong Yantai project (feasibility study)
 - Two ACP100 reactor on the floating platform
 - Electrical generation : 250 MWe



Thanks and Questions

Upcoming Webinars

Date	Title	Presenter
28 September 2022	Development of In-Service Inspection Rules for Sodium-Cooled Fast Reactors Using the System Based Code Concept	Dr. Shigeru Takaya, JAEA, Japan
26 October 2022	Sodium Integral Effect Test Loop for Safety Simulation and Assessment (STELLA)	Dr. Jewhan LEE , KAERI, Republic of Korea
28 November 2022	Visualization Tool for Comparing Energy Options	Dr. Mark Deinert, Colorado School of Mines, USA

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