



Generation IV International Forum

John E. Kelly

**Deputy Assistant Secretary for Nuclear Reactor Technologies
Office of Nuclear Energy
U.S. Department of Energy**

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■ In 1999, low public and political support for nuclear energy

- Oil and gas prices were low

■ USA proposed a bold initiative in 2000

- The vision was to leapfrog LWR technology and collaborate with international partners to share R&D on advanced nuclear systems
- 9 Countries and EU joined USA in developing the initiative
- Oil prices jumped soon thereafter

■ Gen IV concept defined via technology goals and legal framework

- Technology Roadmap released in 2002
 - 2 year study with more than 100 experts worldwide
 - Nearly 100 reactor designs evaluated and down selected to 6 most promising concepts
- First signatures collected on Framework Agreement in 2005; first research projects defined in 2006

“This may have been the first time that the world came together to decide on a fission technology to develop together.”

William Magwood IV, First Chairman of the Generation IV International Forum

■ Sustainability

- Long term fuel supply
- Minimize waste and long term stewardship burden

■ Safety & Reliability

- Very low likelihood and degree of core damage
- Eliminate need for offsite emergency response

■ Economics

- Life cycle cost advantage over other energy sources
- Financial risk comparable to other energy projects

■ Proliferation Resistance & Physical Protection

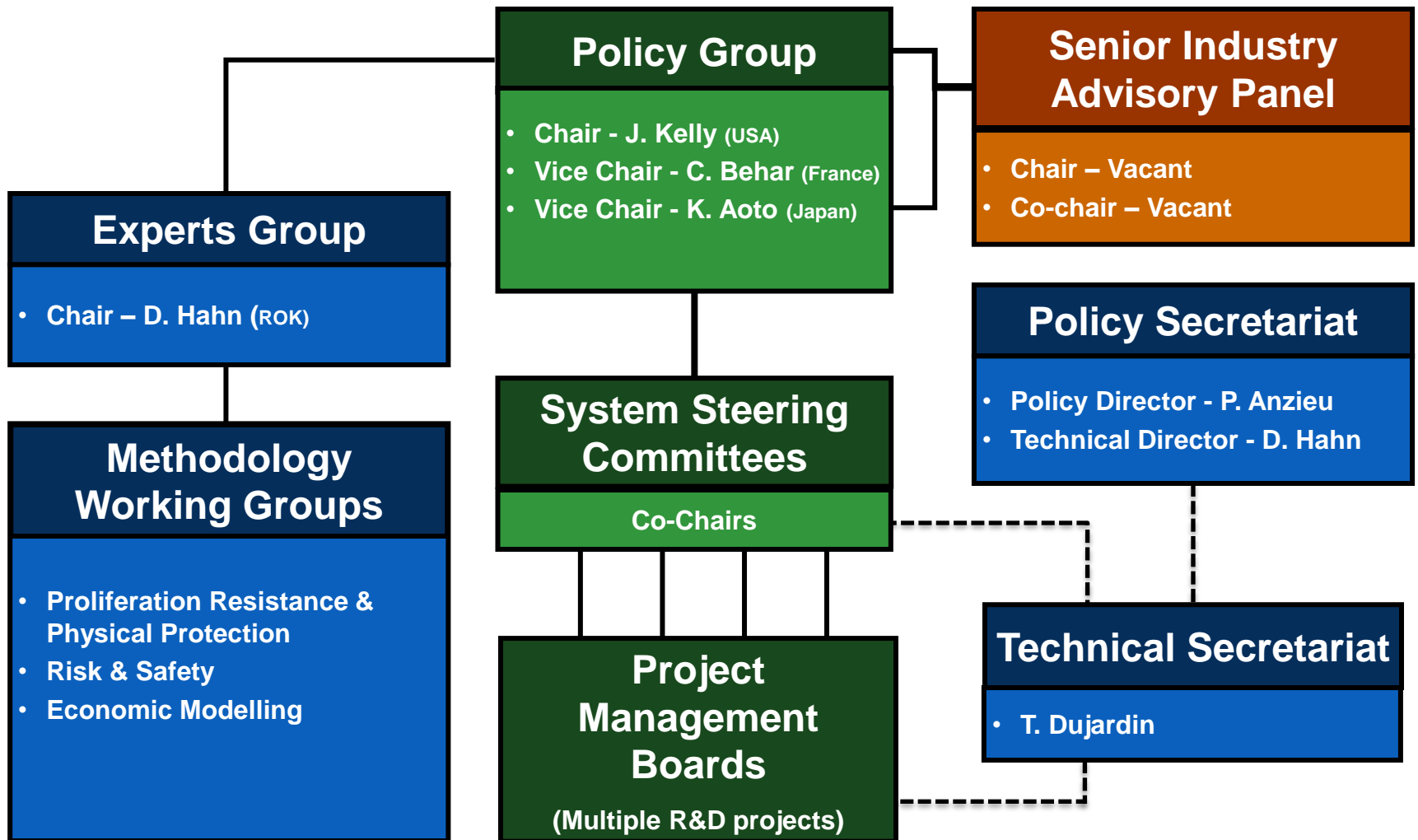
- Unattractive materials diversion pathway
- Enhanced physical protection against terrorism

Thirteen Current Members of Generation IV

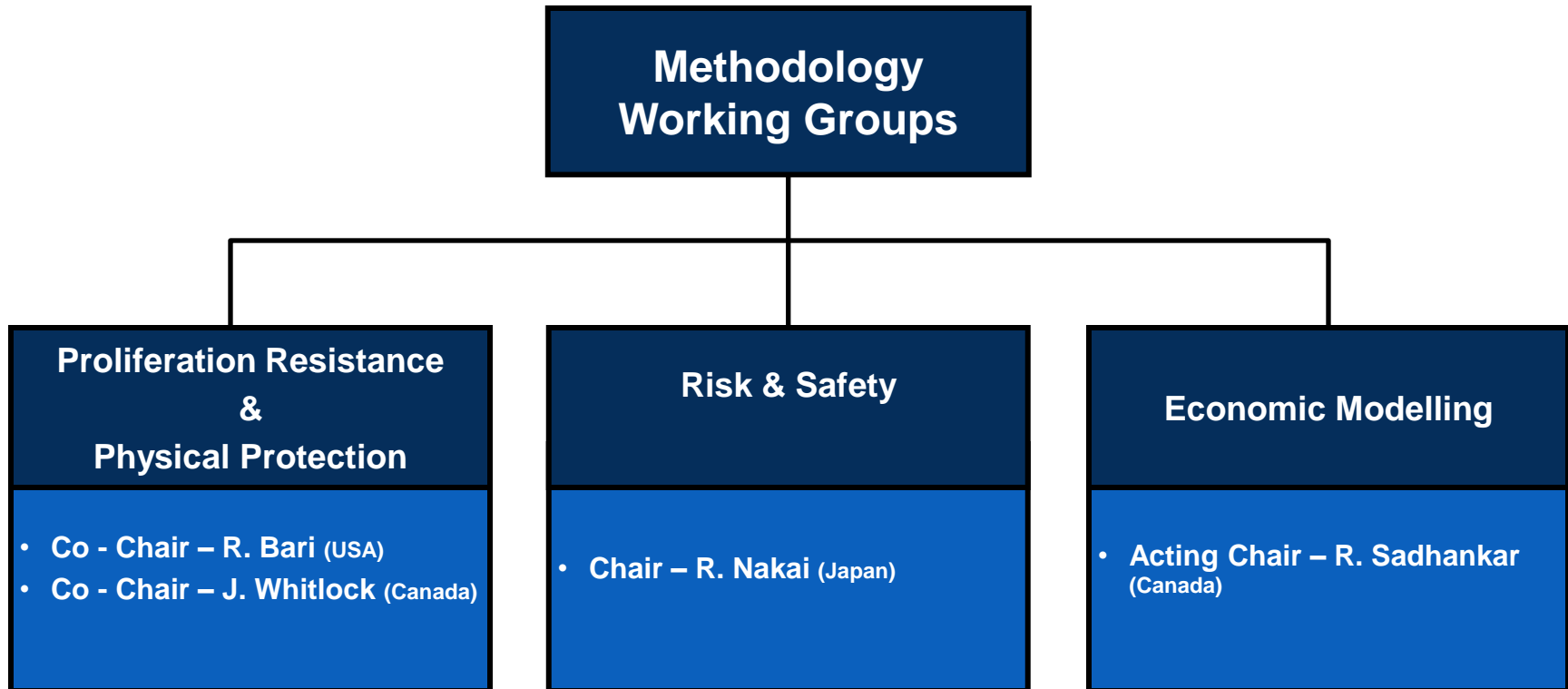
	Argentina*
	Brazil*
	Canada
	People's Republic of China
	Euratom
	France
	Japan

	Republic of Korea
	Russian Federation
	Republic of South Africa
	Switzerland
	United Kingdom*
	United States

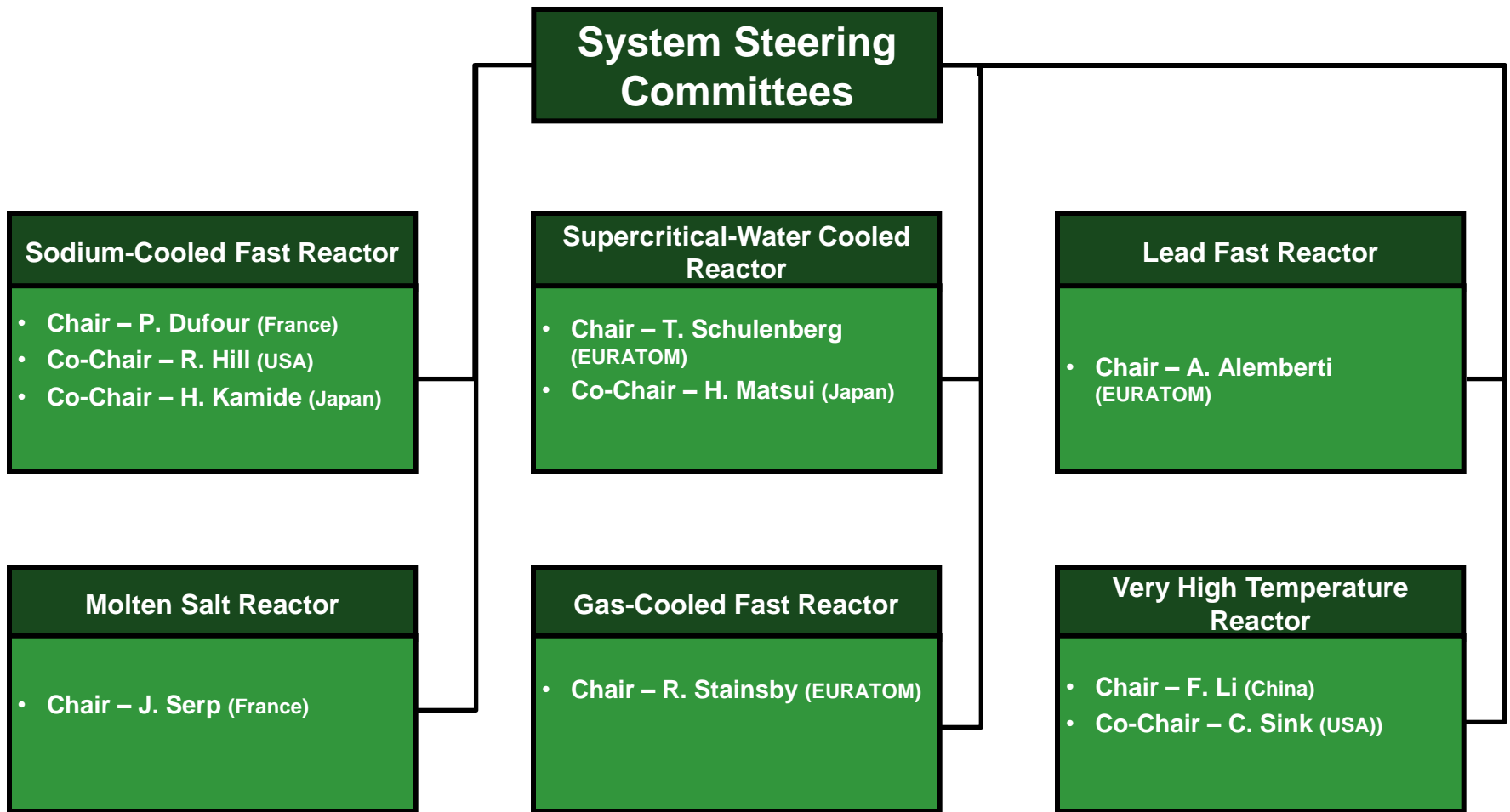
*Non-active member

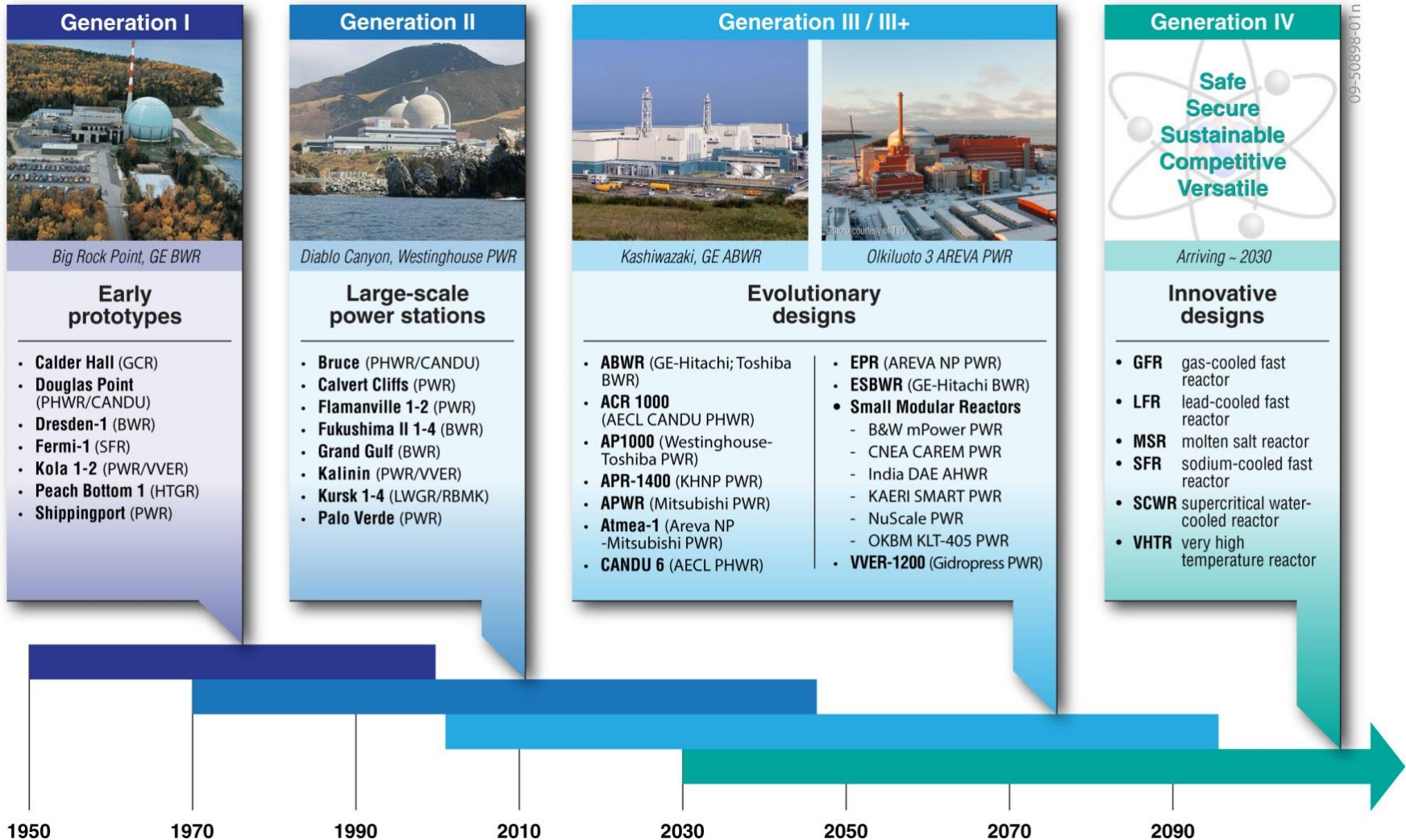


Chairs as of January 2014



System Steering Committees





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■ Integral part of the closed fuel cycle

- Can either burn actinides or breed fissile material

■ Designs being developed

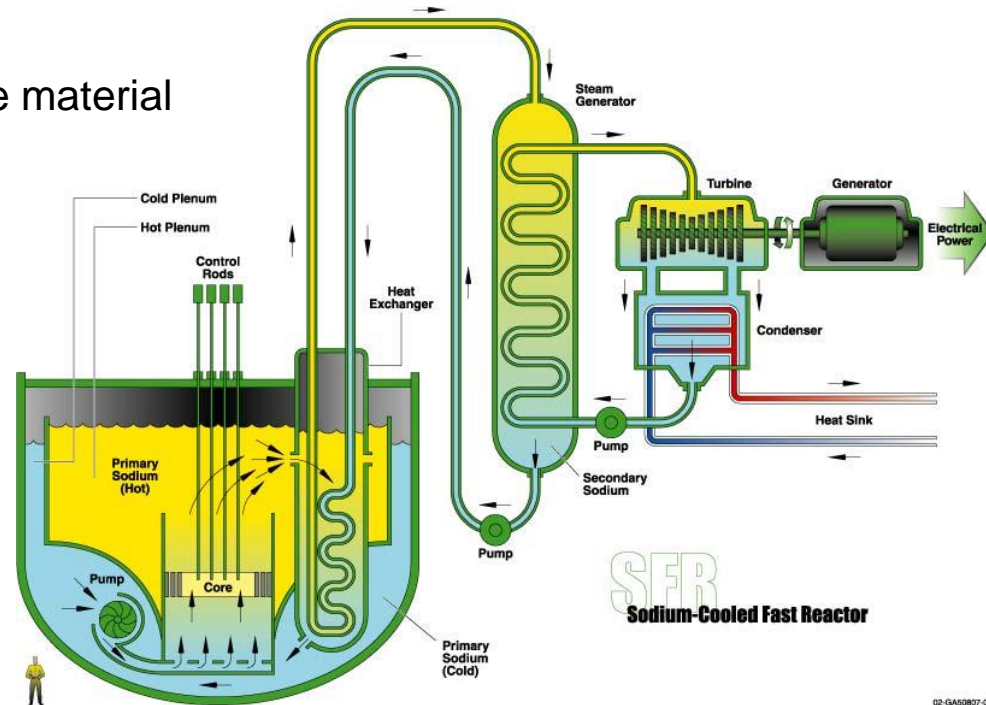
- ASTRID (France)
- JSFR (Japan)
- PGSFR (Korea)
- BN-1200 (Russia)

■ BN-800 (Russia)

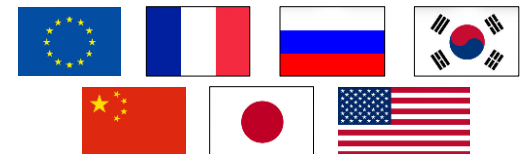
- 2014 - Start-up expected
- 2015 - Fully operational

■ R&D focus

- Analyses and experiments that demonstrate safety approaches
- High burn-up minor actinide bearing fuels
- Develop advanced components and energy conversion systems



500 - 550 °C



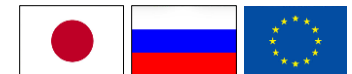
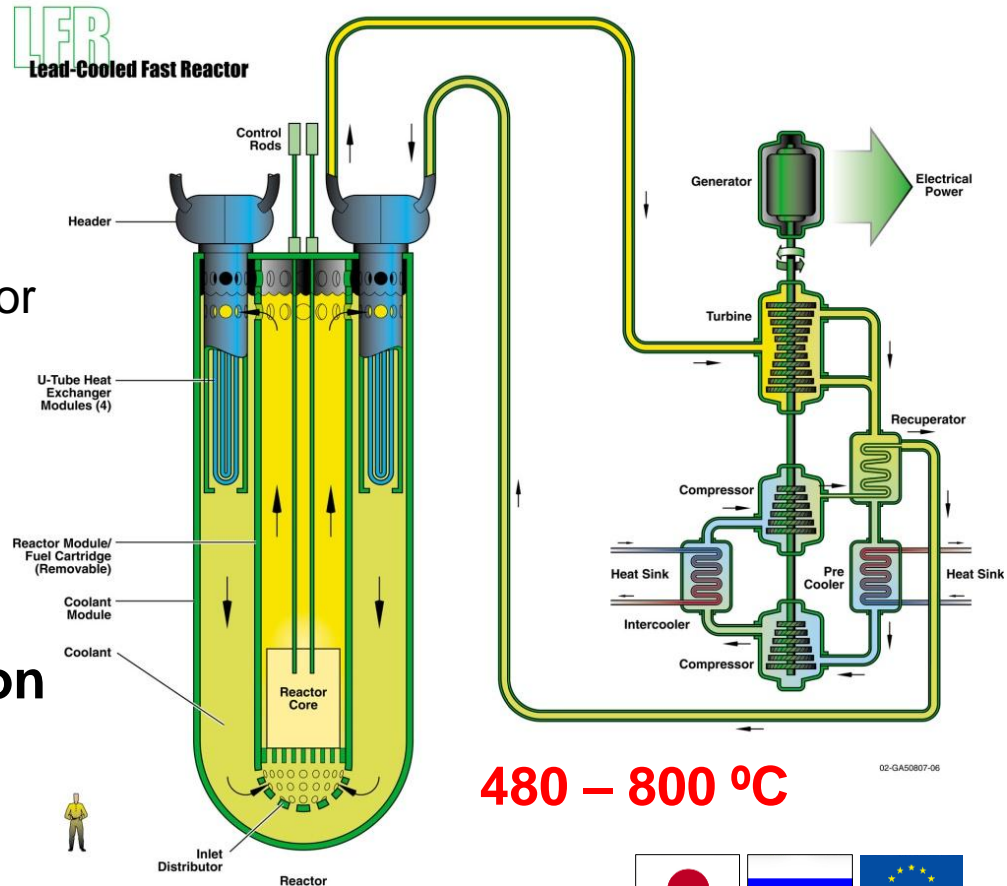
Lead Fast Reactor

■ Lead is not chemically reactive with air or water and has lower coolant void reactivity

■ Three design thrusts:

- European Lead Cooled Fast Reactor (Large, central station)
- Russian BREST-OD-300 (Medium size)
- SSTAR (Small Transportable Reactor)

■ R&D focus on materials corrosion and safety



■ **High temperature, inert coolant and fast neutrons for a closed fuel cycle**

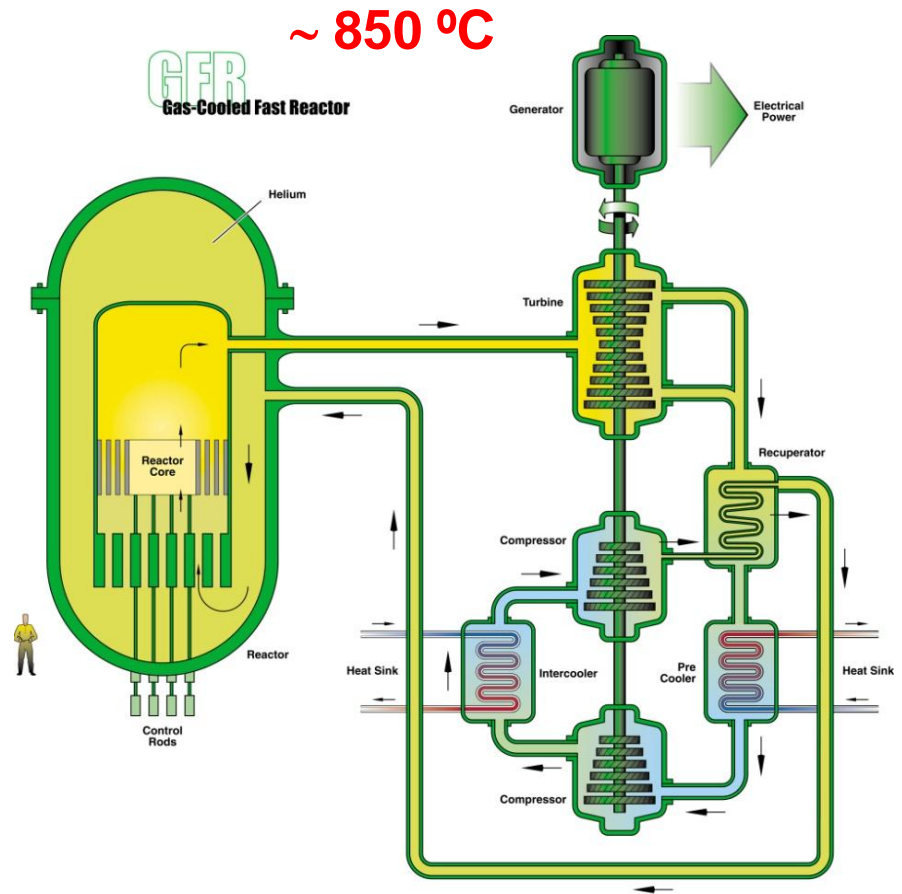
- Fast spectrum enables extension of uranium resources and waste minimization
- High temperature enables non-electric applications
- Non-reactive coolant eliminates material corrosion

■ **Very advanced system**

- Requires advanced materials and fuels

■ **Key technical focus:**

- SiC clad carbide fuel
- High temperature components and materials



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Very High Temperature Reactor

- High temperature enables non-electric applications
- Goal - reach outlet temperature of 1000°C, with near term focus on 700-950° C

- Reference configurations are the prismatic and the pebble bed
 - Designed to be “walk away safe”

- R&D focus on materials and fuels

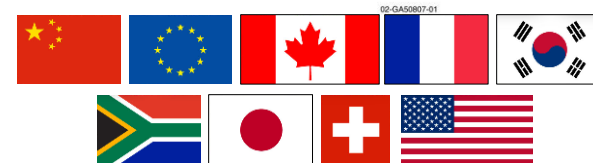
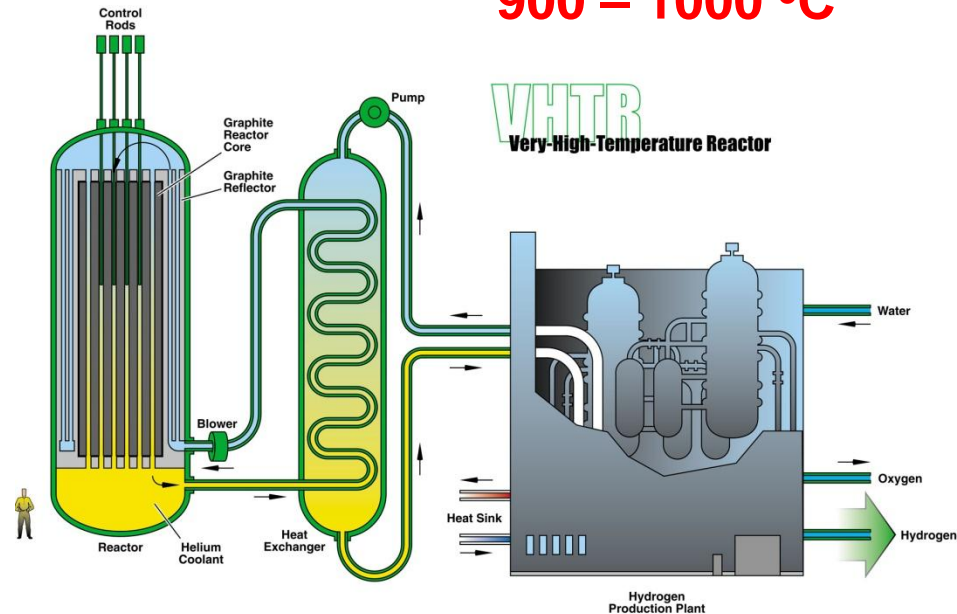
- Develop a worldwide materials handbook
- Benchmarking of computer models
- Shared irradiations
 - Confirmed excellent performance of UO₂ TRISO

- Japan HTTR in operation

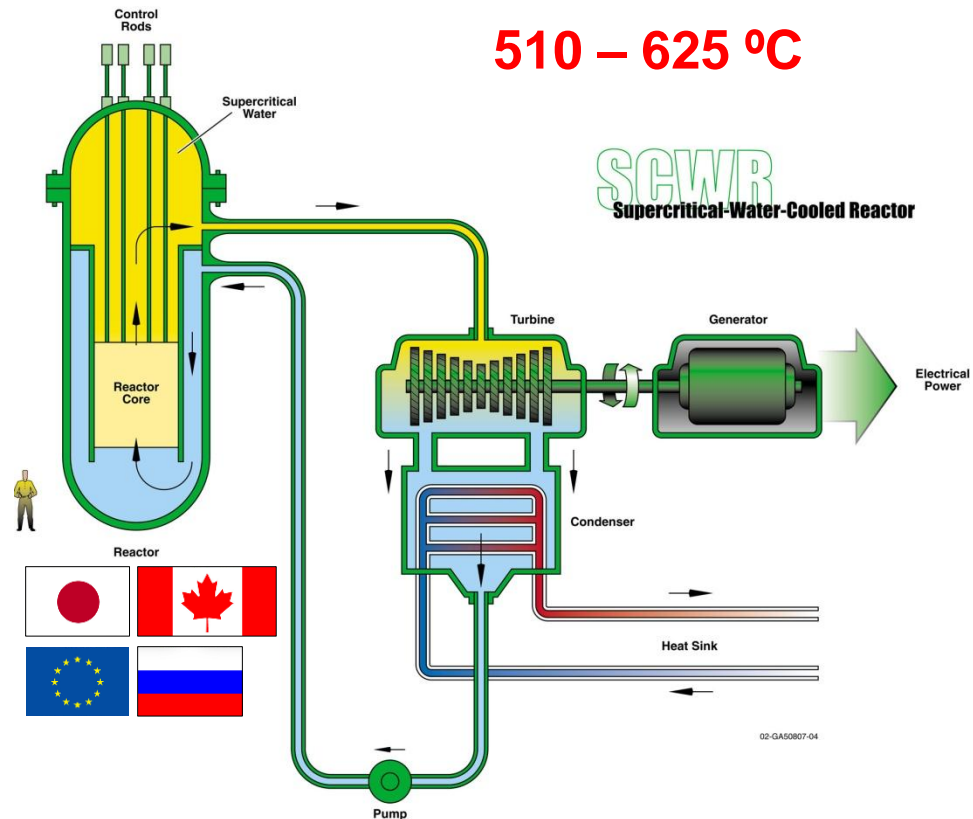
- China HTR-PM demonstration plant under construction

900 – 1000 °C

VHTR
Very-High-Temperature Reactor



- Merges GEN-III+ reactor technology with advanced supercritical water technology used in coal plants
- Operates above the thermodynamic critical point (374° C, 22.1 MPa) of water
- Fast and thermal spectrum options
- Key technology focus:
 - Materials, water chemistry, and radiolysis
 - Thermal hydraulics and safety to address gaps in SCWR heat transfer and critical flow databases
 - Fuel qualification



■ High temperature system

- High temperature enables non-electric applications

■ On-line waste management

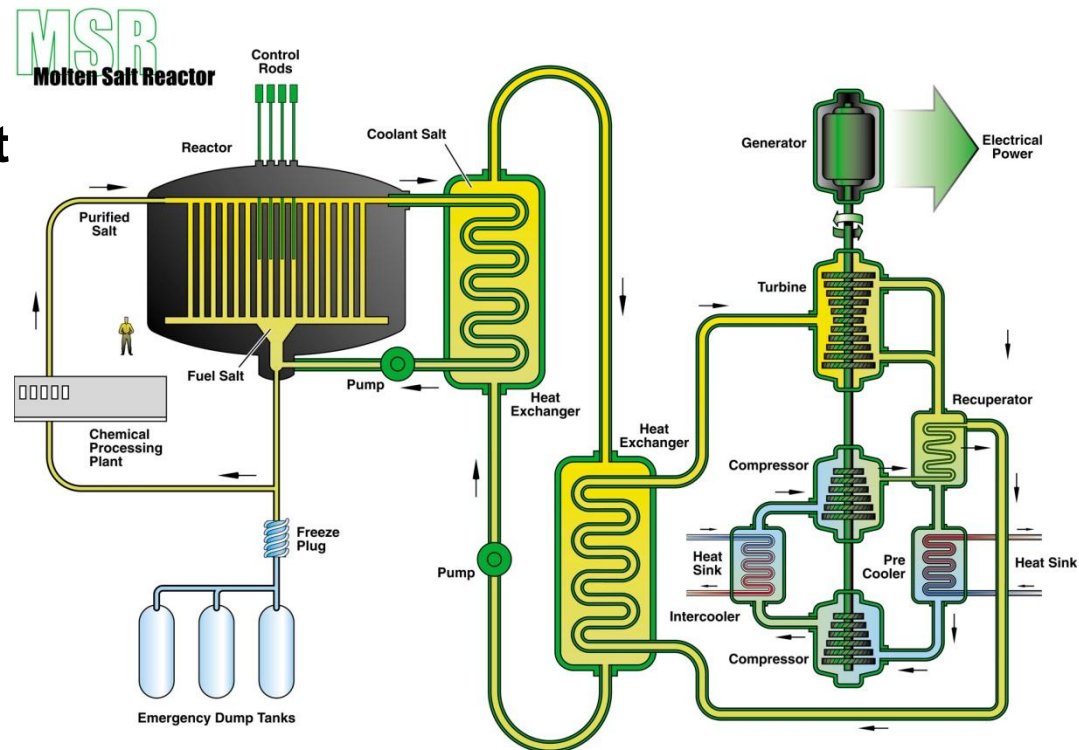
■ Design Options

- Solid fuel with molten salt coolant
- Fuel dissolved in molten salt coolant

■ Key technical focus

- Neutronics
- Materials and components
- Safety and safety systems
- Liquid salt chemistry and properties
- Salt processing











700 – 800 °C



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Generation IV system development in the period through 2013

Generation IV Systems	 Canada	 China	 France	 Japan	 Korea	 Russia	 South Africa	 Switzerland	 USA	 EU
Sodium-cooled Fast Reactor (SFR)		●	●	●	●	●			●	●
Very-high Temperature Gas-cooled Reactor (VHTR)	●	●	●	●	●		●	●	●	●
Gas-cooled Fast Reactor (GFR)			●	●				●		●
Supercritical-water cooled Reactor (SCWR)	●			●		●				●
Lead-cooled Fast Reactor (LFR)				●		●				●
Molten Salt Reactor (MSR)			●			●				●

● Participating member, signatory of a System Arrangement or a Project Arrangement at some point during the period. This table does not necessarily reflect the status of participation as of 1 January 2014.

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- **Over the last decade Gen IV has had major accomplishments**
 - Legal framework established for collaboration
 - Collaborative projects started with significant R&D investment worldwide
 - Prototype demonstrations are being designed and/or built
 - SFR (France and Russia)
 - VHTR (China)
 - **Much still needs to be done before Gen IV systems are a reality**
 - Continue R&D on Gen IV systems
 - Develop advance research facilities
 - Engage industry on the design of Gen IV systems
 - Develop the workforce for the future

GEN IV International Forum

